

Appendix A
Safety Evaluation of the Transfer Cask

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Appendix A

Safety Evaluation of the Transfer Cask

Summary

As stated in the ISF Facility Safety Analysis Report (SAR), Peach Bottom casks PB-1 and PB-2 (also known as CA-SF-006 and CA-SF-005, respectively) will be used to transfer spent nuclear fuel (SNF) from Idaho National Laboratory (INL) dry fuel storage facilities to the ISF Facility. The SNF transfer to the ISF Facility occurs solely within the DOE-ID controlled boundaries of INL over a distance of approximately 500 yards and does not involve the use of public roadways.

The analyses presented in this Appendix demonstrate the Peach Bottom transfer cask's ability to provide geometry control, configuration control, and shielding of the SNF for all credible accidents during transfer from the nearby INTEC to the ISF Facility.

This Appendix is organized in the format of Nuclear Regulatory Commission (NRC) Regulatory Guide 3.48, Revision 1, *Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation or Monitored Retrievable Storage Installation (Dry Storage)*, issued August 1989, and follows closely the main sections and subsections of the ISF Facility SAR.

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1.0 INTRODUCTION AND GENERAL DESCRIPTION OF INSTALLATION

1.1 INTRODUCTION

The ISF Facility to be constructed at the INL is described in the ISF Facility SAR. The purpose of the ISF Facility is to repackage four types of SNF and to store the repackaged SNF until the geological repository is available. The SNF will be transferred to the ISF facility using two existing Peach Bottom casks (PB-1 and PB-2). The purpose of this Appendix is to demonstrate that the Peach Bottom casks meet the requirements of 10 CFR 72 for use as transfer casks. These casks will be used to transfer the four SNF types described below:

- a) Peach Bottom high-temperature gas-cooled reactor (HTGR) Core 1
- b) Peach Bottom HTGR Core 2
- c) Shippingport light-water breeder reactor (LWBR) reflector modules and loose rods
- d) Training, Research, Isotope Production, General Atomics (TRIGA) aluminum clad and steel clad fuel

Section 3.1.1 of the ISF Facility SAR provides detailed physical and historical descriptions of the four SNF types. Figures A-1 through A-4 provide a pictorial description of the fuel transfer process.

When the ISF Facility is completed and operational, the SNF identified above will be transferred to it from INTEC using the Peach Bottom casks. The SNF transfer will occur over a distance of approximately 500 yards along INL site roads. No public highways will be affected. Should an off-normal or accident event occur during transfer of a loaded Peach Bottom cask to the ISF Facility, the cask and contents can be returned to the appropriate INTEC facility for unloading and inspection, as appropriate.

The fuel has either been placed, or will be placed, in one of several different fuel type-specific containers prior to being loaded in the Peach Bottom casks. Following receipt of the transfer cask at the ISF Facility, the cask will be up righted, removed from its transportation trailer, and placed in the ISF Facility cask trolley. Using the trolley, the cask will be moved underneath the Fuel Packaging Area (FPA) fuel transfer port, the cask lid removed, and the fuel containers removed. After the fuel is removed, the lid will be replaced and the Peach Bottom cask will then be moved back to the receipt area, placed on the transportation trailer and returned for reuse. This process is described in detail in Chapter 5 of the ISF Facility SAR.

The Peach Bottom casks are used only on the DOE INL site under DOE-ID's transportation and packaging program. They are not licensed under either 10 CFR 72 or 10 CFR 71, although both casks were fabricated under an NRC Certificate of Compliance (USA/6375/B()F) in the 1970's. DOE-ID has developed analyses that have been used to bound the ISF Facility receipt and handling of the DOE-ID Peach Bottom casks. The analyses and documents provided by DOE-ID include:

- a) drop scenarios for the PB Casks and fuel containers (Ref A-2 through A-6 and A-8)
- b) dose calculations for SNF drop accidents (Ref A-7)
- c) shipping configurations for the SNF (Ref. A-9 through A-12)

- d) criticality safety evaluations (Ref A-13 and A-14).

This document constitutes Appendix A of the ISF Facility SAR, and each chapter follows the same overall format as the SAR.

1.2 GENERAL DESCRIPTION OF INSTALLATION

1.2.1 Arrangement of Major Structures and Equipment

Details on the ISF Facility are provided in the ISF Facility SAR, Chapter 1. Information specific to the Peach Bottom casks and the transportation trailer is provided below.

Figures A-5 and A-6 depict the PB-2 cask and its subcomponents. Figure A-7 depicts an alternate cask lid that is used by DOE-ID for the Peach Bottom core 1 fuel transfers. Drawings for the PB-1 cask are not available from DOE-ID; however, the two casks have been confirmed to be nearly identical, with the exception of the distance between the top and bottom pairs of trunnions. DOE-ID performed field verification of the PB-1 and PB-2 cask configurations in conjunction with DOE-ID's preparation of the cask safety analyses and the results of these field measurements are provided on Figures A-32 through A-34.

Figures A-8 through A-10 document the DOE-ID design for a liner/overpack insert to be used for the Peach Bottom Core 2 and TRIGA fuels. These fuels are stored at the Irradiated Fuel Storage Facility (IFSF), located within the Idaho Nuclear Technology and Engineering Center (INTEC), in unsealed 18-inch diameter containers. Since the IFSF canister diameter is smaller than the cask inner cavity, an inner liner/overpack is required to center and contain the IFSF canister within the Peach Bottom cask.

Figure A-11 illustrates DOE-ID Trailer No. 71808 and its subcomponents. Trailer No. 71808 is a flatbed-type transporter. This transporter will be used to transfer only the PB-1 cask because of the difference in trunnion locations between the two casks.

Figures A-12 and A-13 illustrate DOE-ID Trailer No. E-71801, a 40-ton low-bed, or lowboy-type, trailer and its subcomponents. This transporter can accommodate either a PB-1 or the PB-2 cask since the location of its front trunnion supports is adjustable to meet the dimensional requirements of either cask.

The Interface Control Diagrams (Figures A-1 through A-4) depict the relationship between the casks and the transporters.

1.3 GENERAL SYSTEMS DESCRIPTION

1.3.1 Storage Systems

Not applicable to the Peach Bottom transfer cask.

1.3.2 Transfer Systems

1.3.2.1 Transfer Cask

The PB-1 and PB-2 casks will be used for onsite transfer of SNF from dry fuel storage facilities at the INTEC to the ISF Facility. These casks provide shielding and protection from potential hazards. The casks were designed for a maximum payload weight of 10,000 pounds. The PB-2 cask and its subcomponents are depicted in Figures A-5 through A-7. The casks were originally designed to provide sufficient shielding to ensure that dose rates resulting from shipping the Peach Bottom fuels were maintained below 10 CFR Part 71 regulatory limits. Four lifting trunnions are provided for handling the casks at the ISF Facility. The upper two trunnions are intended for lifting the casks; the lower two trunnions are used to pivot the casks about the lower cask cradle on the trailers. Additional design details of the casks are provided in Section 4.7 of this Appendix.

1.3.2.2 Transfer Equipment

Two transport vehicles are planned to be used for onsite transfer of the Peach Bottom casks. Trailer No. 71808, a flatbed trailer (Figure A-11), or Trailer No. E-71801 (Figures A-12 and A-13) can be used to transport either PB-1 or PB-2.

The petroleum fuel content of the transporter tractor used to place the trailers in the ISF Cask Receipt Area will be administratively controlled to minimize the consequences of fire. Refer to the ISF Facility SAR, Chapters 4 and 8.

1.3.3 Auxiliary Systems

Not applicable to the Peach Bottom transfer cask.

1.3.4 System Operation

Transfer system operations for the four SNF types are depicted in the Interface Control Diagrams (Figures A-1 through A-4).

1.3.5 Arrangement of Storage Structures

Not applicable to the Peach Bottom transfer cask.

1.4 IDENTIFICATION OF AGENTS AND CONTRACTORS

The DOE-ID site support contractor is responsible to DOE-ID for maintaining the Peach Bottom casks. The support contractor is also responsible for loading the cask with SNF at INTEC, transferring the cask to the ISF Facility, and returning the cask to the INTEC after the fuel has been unloaded.

1.5 MATERIAL INCORPORATED BY REFERENCE

This SAR appendix is self-contained and does not incorporate any topical reports or documents submitted in other applications by reference.

2.0 SITE CHARACTERISTICS

This topic is addressed in the ISF Facility SAR Chapter 2.

3.0 PRINCIPAL DESIGN CRITERIA

3.1 PURPOSE OF INSTALLATION

This topic is addressed in the ISF Facility SAR Section 3.1.

3.2 STRUCTURAL AND MECHANICAL SAFETY CRITERIA

The Transfer Cask is classified Important to Safety (ITS). Structural and mechanical transportation safety criteria are established for geometry control, configuration control, confinement barrier, and shielding of the SNF to ensure criticality safety and radiation protection are maintained. Postulated accidents associated with movement of the transfer cask within the ISF Facility are considered to be of low probability and consequences due to the slow speed of travel, short travel distance, and administrative controls and procedures that will be in place during cask movement. Transfer accidents within the ISF Facility are bounded by the analyses that addresses the transfer from the fuel storage location within INTEC to the ISF Facility. The PB Cask designs, evaluations and procedures are summarized or referenced in the appropriate sections of this Appendix. These summaries or references demonstrate that the PB casks provides geometry control, configuration control, confinement barrier, and shielding of the SNF for all credible accidents within the ISF Facility during transfer from the nearby INTEC facility.

3.3 SAFETY PROTECTION SYSTEMS

3.3.1 General

This topic is addressed in ISF Facility SAR, Chapter 3.

3.3.2 Protection by Multiple Confinement Barriers and Systems

3.3.2.1 Confinement Barriers and Systems

Confinement barriers related to the Peach Bottom cask are discussed below. Additional discussion of ISF confinement boundaries is provided in Chapter 3 of the ISF Facility SAR.

3.3.2.1.1 Existing DOE-ID Transfer Cask

The Transfer Casks serve as the SNF confinement barriers and system from the time the transfer cask arrives at the ISF site boundary until the transfer cask lid is removed and the fuel containers are lifted from the cask into the ISF Facility Fuel Packaging Area (FPA).

The SNF containers are loaded into the Peach Bottom cask at the INTEC for transfer to the ISF Facility. After loading the container in the cask, the cask lid is bolted in place. Within the ISF Facility the confinement barriers provided by the Peach Bottom cask are protected by the following facility design criteria and features:

- a) The Transfer Cask is handled with a single-failure-proof crane and lifting device while in the Cask Receipt Area. These components meet the applicable requirements of NUREG-0554 (Ref. A-18), NUREG-0612 (Ref A-19), and ANSI N14.6 (Ref A-20). This crane and its supports have been designed to withstand credible accident loads, including seismic forces. Design criteria associated

with the Cask Receipt Area crane are discussed in detail in Chapters 3 and 4 of the ISF Facility SAR.

- b) The cask is moved within the ISF Facility using a cask trolley that meets the applicable requirements of NUREG-0554, NUREG-0612, and ANSI N14.6 for the control of heavy loads. This trolley has been designed to restrain the cask and prevent overturning under credible accident conditions, including seismic loads. Design criteria associated with the cask trolley are discussed in detail in Chapters 3 and 4 of the ISF Facility SAR.
- c) The structure of the ISF Facility Transfer Tunnel provides protection from credible accident conditions, including seismic events, and tornado winds and missiles. Design criteria associated with ISF Facility reinforced concrete structures are discussed in detail in Chapters 3 and 4 of the ISF Facility SAR.

The direct dose through the composite walls and lids of the Transfer Cask has been considered in the estimate of the overall dose for workers at the ISF Facility. For the cask, worker dose was estimated by a combination of the amount and type of fuel in a given shipment, the internal packaging configuration and self-shielding within the cask, and the time between arrival of the cask at the ISF Facility, unloading of the cask from the heavy haul trailer onto the cask trolley, and placement of the cask trolley inside the Transfer Tunnel. The overall dose associated with ISF Facility operations is discussed in Section 7.4.1 of the SAR. A summary of the occupational doses for ISF Facility fuel handling operations, which includes transfer cask operations, is provided in Table 7.4-2 of the SAR.

Damage caused by postulated accidents and natural phenomena, related to the confinement barrier provided by the Peach Bottom cask are considered to be bounded by the Peach Bottom cask and fuel container drop analyses (Ref A-2 through A-6). A discussion of the credible Transfer Cask accidents is presented in Section 8.2.2 of this Appendix.

Dose calculations have been performed to predict the radiological consequences resulting from a SNF drop accident (Ref A-7). This evaluation used the Radiological Safety Analysis Computer Program, RSAC-6 (Ref A-21), to calculate the dose for each accident scenario. Each of the four fuel types was evaluated and the bounding dose rate at the ISF Facility Controlled Area Boundary (13,700 meters) was associated with the dropping of a PB Cask containing 18 Peach Bottom Core 1 elements.

In the dose evaluations for this bounding analysis, the following were assumed:

- a) 25% of the fuel is assumed to have fractured or broken to the point that powder exists in the fuel following the drop event.
- b) An airborne release fraction of 0.1% is assumed to have become airborne from the fuel that has broken into powder.
- c) The Peach Bottom cask was assumed to further limit the airborne release to 0.1%
- d) The accident is assumed to occur outside at ground level
- e) Class F meteorological stability class with a 1.04 m/s wind speed is used
- f) A breathing rate of $3.33 \times 10^{-4} \text{ m}^3/\text{s}$ is used
- g) The individual at the Controlled Area Boundary remains for the entire plume duration.

The highest calculated dose was 2.19×10^{-5} rem TEDE which remains below the limits defined in 10 CFR 72 (Ref. A-15) for normal, off-normal and accident conditions.

3.3.3 Protection by Equipment and Instrumentation Selection

3.3.3.1 Equipment

Key Transfer cask equipment that provides protection or confinement barriers to the SNF is listed in Table A3.3-1. Principal design criteria are also presented in this table. Additional design criteria and further discussions of subsystems and components are provided in Chapters 4, 5, and 8 of this Appendix. Other key equipment that provides protection of the SNF in the ISF Facility is discussed in the ISF Facility SAR, Section 3.3.

3.3.3.2 Instrumentation

The Peach Bottom casks are each fitted with four quick disconnect fittings. These are located within the Peach Bottom cask trunnions (see Figure A-5). A pressure gauge may be inserted into the disconnect fittings to determine if the cask has become pressurized during transfer.

3.3.4 Nuclear Criticality Safety

Nuclear criticality safety criteria are discussed in Section 3.3.4 of the ISF Facility SAR.

Criticality safety evaluations that have been performed (Reference A-13 and A-14) bound the fuel types and shipping configurations that will be used to transfer the SNF to the ISF Facility. Section 4.7.3.4 of this Appendix provides a summary of the results of these analyses.

3.3.5 Radiological Protection

This topic is addressed in the ISF Facility SAR.

3.3.6 Fire and Explosion Protection

This topic is addressed in the ISF Facility SAR.

3.3.7 Materials Handling and Storage

Not applicable to the Peach Bottom cask.

3.3.8 Industrial and Chemical Safety

Not applicable to the Peach Bottom cask.

3.4 CLASSIFICATION OF STRUCTURES, COMPONENTS, AND SYSTEMS

The Peach Bottom Cask has been determined to be an Important to Safety system using the criteria presented in Chapter 3 of the ISF Facility SAR.

3.5 DECOMMISSIONING CONSIDERATIONS

Decommissioning is addressed in the ISF Facility Proposed Decommissioning Plan (Ref A-17).

4.0 INSTALLATION DESIGN

4.1 SUMMARY DESCRIPTION

A summary of the ISF Facility installation design is addressed in Section 4.1 of the ISF Facility SAR. A description of the Peach Bottom transfer cask is discussed in Section 4.7.1.1.

4.2 STORAGE STRUCTURES

Not applicable to the Peach Bottom transfer cask.

4.3 AUXILIARY SYSTEMS

Not applicable to the Peach Bottom transfer cask.

4.4 DECONTAMINATION SYSTEM

Methods and systems for decontamination of the Peach Bottom casks at the ISF Facility are addressed in the ISF Facility SAR, Section 4.4.

4.5 TRANSFER CASK REPAIR AND MAINTENANCE

The ISF Facility does not have shipping cask repair or cask maintenance facilities. Repair and maintenance of this government furnished equipment will be performed by DOE-ID through its support contractor at separate facilities. Repair and maintenance of the ISF Facility's cask lifting hardware is addressed in the ISF Facility SAR, Section 5.1.3.5.

4.5.1 Routine Inspection

Routine inspection requirements for the Peach Bottom cask will be performed in accordance with DOE-ID's established inspection and maintenance program. Generic items for routine inspection include:

- a) Visually inspect the cask exterior for cracks, dents, gouges, tears, or damaged bearing surfaces. Particular attention is to be paid to the cask trunnions.
- b) Visually inspect all threaded parts and bolts for burrs, chafing, distortion, or other damage.

4.6 CATHODIC PROTECTION

Not applicable to the Peach Bottom transfer cask.

4.7 SPENT FUEL HANDLING OPERATION SYSTEMS

4.7.1 Structural Specifications

Details on structural specifications for structures, systems and components (SSC) that interface with the Peach Bottom cask are provided in Chapter 4 of the ISF Facility SAR. The following sections focus on the structural specifications for the Peach Bottom cask.

4.7.1.1 Peach Bottom Casks

The Peach Bottom casks were fabricated and maintained, under Certificate of Compliance (COC) USA/6375/B()F (Ref A-23) for over-the-road shipment of radioactive material in accordance with 10 CFR 71. The COC for these casks expired on September 30, 1979. Figures A-5 and A-6 are the assembly drawings for the PB-2 cask. The PB-1 and PB-2 configurations were field verified in conjunction with preparation of the cask safety analyses and the results of these field measurements are provided on Figures A-32 through A-34.

Analyses has been performed to demonstrate that the PB-1 and PB-2 transfer casks will survive credible handling accidents associated with the transfer of each of the four SNF from the nearby INTEC (Reference A-2 through A-6, A-13 and A-14). These analyses, coupled with the discussion provided in Chapter 8 of this Appendix, demonstrate that the Peach Bottom casks will provide the necessary geometry control, configuration control, confinement barrier, and shielding of the SNF during transfer operations to ensure that radiation protection and criticality safety requirements of the ISF Facility are met.

Section 8.1 of this Appendix describes the off-normal events analyzed, while Section 8.2 describes the credible accident scenarios associated with receipt and handling of the Peach Bottom cask at the ISF Facility.

Codes and Standards

Based on information contained in the original Safety Analysis Report submitted to the NRC in 1970 (Ref. A-16) resulting in the issuance of the COC (Ref A-23), the Peach Bottom casks welding procedures were “essentially those of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (B&PV) Code Section VIII, Unfired Pressure Vessels, 1962 Edition, and Section IX, Welding Qualifications, 1962 Edition.”

Materials of Construction - Original Configuration

The following configuration information is from the original safety analysis report (Ref. A-16) submitted to the NRC for licensing of the casks and information contained in the Safety Analysis Report for INL on-site shipments using the Peach Bottom cask (Ref A-26). The original design layout of the PB-1 shipping cask for transporting irradiated Peach Bottom 1 reactor fuel elements is shown in Figure A-5. The cask had a calculated loaded weight of 62,800 pounds when transporting Peach Bottom fuels. When transporting the maximum contents payload of 10,000 pounds, the cask had a calculated loaded weight of 67,100 pounds. It has an outer diameter of 42.5 inches, and an overall length of 191.12 inches, including impact limiters, and a width across the trunnions of 50.0 inches. The cask internal cavity is 26 inches diameter and 159 inches long. Canned fuel elements were positioned within the cavity in fuel element baskets.

The cylindrical cask body was constructed with a 0.25-inch, Type 304 stainless steel cavity liner, a maximum of 6.25 inches of lead, a 1.50-inch mild steel outer shell, and a 0.25-inch Type 304 stainless steel overlay. The cavity liner was seam-welded and polished to a No. 3 finish. It was welded at both ends to offset cones, which formed cavities for the end closures. The lead is 5.25 inches thick from the bottom of the cavity to 24.5 inches above the bottom; it is 6.25 inches thick from 24.5 inches above the bottom to 134.5 inches above the bottom; and it is 5.25 inches thick over the remainder of the length. Since lead

shrinks upon solidification, a patented fin arrangement was used to attach the lead to the inside of the outer shell. The fins bridged the gap between the lead and outer shell and enhance the transfer of heat. A venting device for the head cavity prevents a buildup of excessive pressure from either moisture or lead during a fire-temperature excursion. The stepped outer shell was constructed by welding three coaxial, formed and welded, mild steel cylinders. The overlay was welded to the outer shell at the end of each cylinder, and at cutouts around each trunnion. It was spaced from the outer shell by 1/16-inch spot welded spacers on the outer shell, and served as a heat shield to inhibit lead melting during the hypothetical fire-temperature excursion. A pressure relief plug in the overlay shell prevented a buildup of excessive pressure from moisture.

Guide pins provided final alignment of the cover bolt holes with tapped holes in the ends of the cask body. Twelve 1.25-inch diameter ASTM A325, cadmium-plated steel bolts secured each cover. A silicone-rubber O-ring gasket seal was originally used between the cask seat at each end to provide a secondary containment system for the cask contents during shipment.

Heat rejection is accomplished by conduction through the cask walls and radiation and convection from the cylindrical wall of the cask. The cask was designed to operate either wet or dry. For the purpose of SNF transfers to the ISF Facility, all shipments will be dry.

Four 8-inch diameter lifting and pivoting trunnions were welded to the outer shell. Each trunnion is approximately 4.5 inches in length. A 0.5-inch thick Type 304 stainless steel patch plate was welded to the outer shell at each trunnion for added strength, resulting in an effective trunnion length of 4 inches. The trunnions permit raising the cask from the horizontal to the vertical position and vice versa with minimum effort. They also served as a means of attaching the cask to the transport vehicle-mounting cradle. The trunnions are hollow and provided protective housings for the drain valves, flush valve, pressure gauge, pressure-relief valve, and valve exhaust filter. The pressure relief valve was set at the seal test pressure of 100 psig. Pipe plugs were used as seals for the pressure gauge, vent and drain lines. The open end of the trunnion was covered with a 9-inch diameter, 0.5-inch thick plate attached with six fasteners.

The cask was mounted horizontally and handled during transport using all four trunnions in a structural steel cradle that was designed to spread the load. Trunnion sockets on the vehicle-mounting cradle support and secure the cask trunnions. They allowed rotation of the cask from the vertical to the horizontal shipping position. One set of trunnion sockets was adjustable to accommodate changes in the length of the cask due to temperature changes. Pads on the vehicle-mounting cradle provided additional support to the cask when it was in the horizontal position.

An impact limiter was attached to each end with four of the twelve 1.25-inch cover bolts in order to limit the impact load on the fuel canisters during an accidental drop. The impact limiters were constructed by welding a bundle of 2.5-inch nominal diameter by 18 gauge mechanical tubing between 0.25-inch Type 304 stainless steel plates and enclosing the bundle with a 0.0625-inch Type 304 stainless steel shell as shown in Figure A-5. A 4-inch long skirt extended from the impact limiter over the cask for added resistance to radial motion in a corner drop.

The top and bottom lids had 4 inches of lead sandwiched between two 1.50-inch Type 304 stainless steel plates. The upper lid had a removable center plug, which allowed lifting rods to pass through for lifting and supporting fuel bundles/canisters. This feature will not be used at the ISF Facility. The cask lids are tapered to allow easier alignment in the conical recessed cask openings. The lids were secured to each end

of the cask with twelve bolts. Eight of the bolts fit into recesses in the cask lid, and the remaining four bolts were used to attach the external impact limiters to the lids but also thread into the cask body. Four other holes were also provided in the lids: two holes located 180 degrees apart, for seating of the cask guide pins in the lids; and two holes located 180 degrees apart for attachment of lifting rings. The lids are handled by a two-point pickup.

Transfer Cask Configuration for SNF Transfers to the ISF Facility

Impact limiters will not be used due to the short transfer distance to the ISF Facility, except possibly in the case of the TRIGA fuel transfers. In addition, O-ring seals will not be used during fuel transfers to the ISF Facility. A drop evaluation addressing the configuration without impact limiters (Reference A-2) has been performed. Additionally, for shipments of Peach Bottom Core 1 fuel canisters, the top lid of the cask will be replaced. The replacement lid will be 6.25-inch thick, constructed of A36 carbon steel (Figure A-7). For the fuel transfers to the ISF Facility, the transfer casks will utilize ASTM A-276, UNS 21800 (Nitronic 60) bolts for the lids. Where impact limiters are not used, four new, shorter bolts may be installed in the cover in place of those that originally held on the impact limiters.

Table A4.7-1 contains nominal design dimensions and weights for the two Peach Bottom casks for both the original and modified configurations. Table A4.7-2 contains information on the cask materials of construction. Table A4.7-3 contains information on the mechanical properties of the cask materials. Field measurements taken of the Peach Bottom casks can be found on Figures A-32 through A-34.

Fabrication and Inspection

Based on information in the original Safety Analysis submitted to the NRC in 1970, the Peach Bottom shipping casks were fabricated and inspected to the following Battelle Memorial Institute (BMI) procedures and specifications:

- Cask Surface Finish Procedure 748-SF-801
- Welding Procedure for Cask Construction Procedure 748-WP-101 (“essentially those of the American Society of Mechanical Engineers (ASME) Boiler and Pressure (B&PV) Code Section VIII, Unfired Pressure Vessels, 1962 Edition, and Section IX, Welding Qualifications, 1962 Edition”). Based upon a review of the original Safety Analysis prepared by BMI, the 10 CFR 71 Certificate of Compliance (Certificate Number 6375) issued by the NRC, and drawings prepared by BMI, a list of requirements from cited Codes and Standards was developed. Compliance or exceptions to these requirements were also demonstrated. This review identified four specific exceptions to the ASME Code. These four exceptions were with respect to the above welding procedure (two material specifications and two electrode specifications that were not listed in the 1962 Edition of ASME). These exceptions, their justifications, and compensatory measures are provided in Table A4.7-4.
- Liquid Penetrant Inspection 748-PT-201
- Hydrostatic Testing Procedure 738-HT-601
- Pouring Procedure for Lead Shielding 748-LP-1001

- Shielding Integrity Testing Procedure 748-GP-401

Features Covered by QA Program

The original construction and fabrication of the Peach Bottom cask was performed in accordance with the construction specifications and procedures described in the original Safety Analysis Report submitted to the NRC for application of a Certificate of Compliance (Ref. A-16). All Quality Assurance activities associated with the Peach Bottom transfer casks, and the SNF to be transferred therein, will be performed under the ISFSI Quality Assurance Program.

4.7.1.2 Inner Containers

The *DOE Standard for Hoisting and Rigging* (Reference A-22) requires that all structural and mechanical “below-the-hook” lifting devices pass a rated load test such that “the rated load capacity shall not be more than 80 percent of the maximum load sustained during the test.” Some of the fuel containers do not have a redundant load path provided, nor have they been demonstrated to meet either the double safety factor criteria identified in the guidance of NUREG-0612, (Reference A-19). For those fuel containers that do not meet the double safety factor criteria, the consequences of a drop event have been evaluated. Section 8.2.2.1 of this Appendix describes the credible accident scenarios associated with receipt and handling of the fuel containers.

Because of the different storage locations and dates of transfer, the fuel elements from each source have been packaged differently. A description of each of the inner containers follows.

4.7.1.2.1 Peach Bottom Core 1 Packaging

Peach Bottom Core 1 fuel assemblies, shown in Figure A-14, were placed in sealed aluminum canisters with stainless steel liners (Figure A-15) at the Peach Bottom Nuclear Station after removal from the reactor. Failed Core 1 fuel assemblies were removed from the reactor with a stainless steel assembly removal tool (ART), and both the tool and fuel element were placed in a sealed canister as shown in Figure A-16. The sealed storage canisters were loaded and sealed in a helium atmosphere and then checked for leaks. If leaks were detected, the entire canister and fuel element were then placed in a second aluminum storage canister (“salvage canister,” Figure A-16). A typical loaded storage canister weighs about 150 pounds.

The fuel canisters were shipped to the INL in the Peach Bottom cask. Up to eighteen elements at a time were positioned in the cask by means of a basket assembly (see Figures A-17 and A-18). At INTEC, an entire basket assembly loaded with fuel canisters was lowered into a below-grade drywell. A loaded basket weighs a maximum of 5150 pounds. Forty-six basket assemblies are situated in individual drywells. Removal and canning of the Peach Bottom Core 1 fuel resulted in a number of package types (fuel canisters) that were loaded into the baskets.

Support Plate for Corroded Peach Bottom 1 Fuel Baskets - Since being placed in existing storage, visual inspections using remote television cameras indicate that water has entered the interior of the storage area, causing corrosion of the aluminum basket bottoms, sides of the baskets, and the tops of the fuel canisters. Fairly large corrosion deposits have been visually recorded on the bottoms of the handling baskets and fuel canister tops. It is assumed that all storage configurations have corroded to some extent. To

structurally reinforce the aluminum baskets for transfer to the ISF Facility, a new support plate will be installed on the bottom of the existing aluminum baskets (new support plate is not required for stainless steel baskets). The support plate prevents fuel cans from dropping through the corroded basket bottoms in the event of failure. The support plate is connected to a rod that extends through the center tube of the basket. Figures A-21 through A-24 illustrate the overall design and details of the support plate and associated hardware. The connection is made using a remote tool to insert the rod through the penetration in the cask top lid, through the center position of the basket, to the new support plate beneath the basket into which it is threaded and torqued. The new rod also has a threaded lifting fixture at the top, to which a lifting rod can be connected for transferring the fuel basket. The basket will continue to be relied upon to provide lateral support for the canned elements. The support plate has a diameter of 25 inches. It is fabricated of 0.75-inch Type 304 stainless steel plate. The support rod is made of 0.625-inch Nitronic 60 and has a length of 153.5 inches.

4.7.1.2.2 Peach Bottom Core 2 Packaging

Peach Bottom Unit 1 Core 2 fuel elements are stored in unsealed steel canisters at the IFSF. The Core 2 SNF was packaged for shipment using canisters of the same type as those used for Core 1. However, instead of placing the fuel into drywells similar to those used for the Core 1 fuel, all the Core 2 fuel was stored in the IFSF. Since the IFSF storage canisters are approximately 11 feet in length and 18 inches in diameter, once the fuel was received at INTEC, the graphite fuel elements were removed from the aluminum canisters and the upper reflector section was cut from the top of each element. The resulting cropped elements were stored up to 12 per IFSF canister (nominal count is 12 per canister), and the canisters were then placed into IFSF dry storage.

There are two designs of IFSF canisters, each 18 inches in diameter and nominally 11 feet long. The canister bodies are 18-inch diameter Schedule 10 pipe. One of these is A53B carbon steel pipe and the other is Type 304L stainless steel Schedule 10S (0.188 in.) pipe. Each of these incorporates a lid with lifting bail although each has a different mechanism for latching the lid.

The carbon steel IFSF canister is shown in Figure A-25. The lid design reduces air exchange and particulate leakage, but does not provide a confinement seal. Figures A-27 and A-28, show the second, thinner-walled stainless steel IFSF canister. Figure A-29 shows a stainless steel canister design in which one and one-half sectioned Core 1 fuel elements will be placed prior to placement in an IFSF canister and transferred to the ISF Facility.

External visual examinations of the IFSF canisters containing the Core 2 fuel have revealed no pertinent exterior corrosion. No internal inspections have been performed. Little or no corrosion is expected since the fuel was received dry from the Peach Bottom HTGR, and has been in dry storage since the time of receipt.

Because the IFSF canisters containing the Core 2 fuel are not sealed and are smaller than the cask cavity, an inner liner and overpack must be provided for the onsite transfers. Figures A-8 through A-10 show the design for this liner and overpack. This liner and overpack is intended for use in conjunction with the Peach Bottom casks when transferring the Core 2 fuel in IFSF canisters.

4.7.1.2.3 Shippingport Fuel Packaging

After irradiation in the Shippingport reactor, the LWBR fuel was shipped to INL. Fuel rods were removed from selected assemblies for examination at the Expended Core Facility (ECF). Following examination, the fuel and pieces were placed in stainless steel storage canisters (Figures A-35, A-36, and A-37). The LWBR storage canisters are constructed of Type 304 stainless steel. The dimensions are 25.5 inches outer diameter by 158 inches outside length. The canister shell is welded to a girth ring, which contains six 1-inch diameter threaded holes for lifting prior to placement of the canister closure head. A crushable pad with a load spreading plate, upon which the LWBR fuel module rests, sits on the bottom plate of the canister. The closure head is secured with twelve 1-inch diameter bolts, and sealed with a metallic O-ring gasket of silver-plated Inconel. Each closure bolt hole is also sealed with a metallic O-ring. All components are constructed of Type 304 stainless steel.

The irradiated reflector modules were loaded into the LWBR storage canisters under water. Before transfer from the ECF to INTEC, all LWBR reflector modules were dried in accordance with approved ECF procedures using a commercial nuclear-type vacuum drying process. No external heating or other condition methods were applied in conjunction with the vacuum process. After dewatering, the canisters were isolated and monitored for increased internal pressure to verify the completeness of water removal. The canisters were then backfilled with neon gas and leak tested. The canisters were then shipped to INTEC for drywell storage. Only the Shippingport reflector modules and a reflector loose rod canister are to be transferred for storage in the ISF Facility.

No external or internal inspections of the LWBR fuel storage canisters have been performed while in storage at INTEC. Therefore, the actual condition of the canisters and fuel is unknown. However, the assumption is that little or no corrosion can occur inside or outside the canisters because of the pre-storage drying process and because they have been in dry storage ever since. The LWBR Zircaloy cladding is robust, similar zircaloy cladding used in most commercial BWR and PWR fuel and has proven resistant to corrosion both in service and in storage. The Shippingport LWBR modules were also placed into stainless steel liners, dried to remove all liquid water, backfilled with neon gas, and sealed. The CPP-749 vaults, in which these liners are stored, are periodically monitored, and no abnormal gases have been detected. However, an actual gas sample from inside the sealed liners has not been taken since the modules were placed in storage. In addition, dry well temperatures and pressures were monitored for a period of time after receipt of the LWBR storage canisters at INTEC. No abnormal indications were observed.

4.7.1.2.4 TRIGA Fuel Packaging

The ISF Facility is designed to store approximately 1600 TRIGA fuel elements. Over half the TRIGA fuel elements that will be transferred to the ISF are currently stored at INL. These fuel elements are currently stored in canisters in the INTEC IFSF (aluminum and stainless steel clad) and in the INTEC-666 underwater storage basin (stainless steel clad only). The TRIGA elements in the underwater storage basin will be placed in dry storage cans and transferred to the IFSF prior to delivery to the ISF Facility.

DOE-ID continues to receive TRIGA fuel from domestic and foreign sources. All TRIGA fuel accepted for storage at the INTEC must meet the DOE's Foreign Research Reactor Fuel Acceptance Criteria (Ref. A-25), which include inspections for structural failures and corrosion. All TRIGA fuel must first be

inspected and accepted for storage at INTEC, prior to all planned transfers from INTEC to the ISF Facility.

In order to facilitate TRIGA fuel loading, transfer, and handling, all stainless steel or aluminum clad TRIGA fuel will be packaged in a standard configuration within the Peach Bottom casks for transfer to the ISF Facility. This will consist of the following can/bucket/canister configuration:

Can: Up to five individual TRIGA SNF elements will be placed in a five position standard TRIGA fuel can (Figure A-38). The pintle used to handle the can is the same as the TRIGA fuel element pintle.

Bucket: Six cans, as described above, will be placed in a TRIGA bucket (Figure A-31 and A-31a). The bucket is approximately 31 inches tall and is handled with a locking hook.

IFSF Canister: Three buckets as described above, will be placed in an IFSF Lighter Weight Storage Canister (Figure A-27 and A-28). In order to maintain configuration control with the Lighter Weight Storage Canister, the following canister gap plug will be utilized.

Canister Gap Plug: A Canister Gap Plug (Figure A-39) will be placed on top of the upper bucket to fill the remaining space in the canister to prevent any significant shifting of the buckets, and preclude/mitigate safety issues when the cask is placed horizontally on the transport trailer for delivery to the ISF Facility. The Canister Gap Plug will have the same lifting fixture, bottom plate, and outer diameter as the TRIGA bucket described above. The weight of the gap plug will be kept to the minimum required to provide the necessary structural integrity and maintain the total loaded weight of the canister below 2000 pounds.

Each shipment will contain up to a maximum of 90 standard stainless and/or aluminum clad TRIGA elements.

As stated previously, the IFSF canisters containing the TRIGA fuel are not sealed and are smaller than the cask cavity. An inner liner and overpack must be provided for the onsite transfers. Figures A-8 through A-10 show the design for this liner. This liner is intended for use in conjunction with the Peach Bottom casks when transferring the TRIGA fuel in IFSF canisters.

4.7.2 Installation Layout

This topic is addressed in the ISF Facility SAR.

4.7.3 Individual Unit Descriptions

4.7.3.1 Functions of Fuel Handling Operational Area

Not applicable to the Peach Bottom transfer cask.

4.7.3.2 Components for Fuel Handling Operations

Not applicable to the Peach Bottom transfer cask.

4.7.3.3 Design Bases and Safety Assurance

Details on the design bases and safety assurances for SSCs that interface with the Peach Bottom cask are provided in Chapter 4 of the ISF Facility SAR. The following sections focus on the design bases and safety assurances for the Peach Bottom cask. Analyses presented are based on information that is relevant to the planned packaging and shipment configurations.

4.7.3.3.1 Structural Evaluation of Cask Trunnions in Accordance with NUREG-0612 Criteria

Purpose

The purpose of this evaluation is to determine the design safety factor with respect to ultimate strength of the Peach Bottom Cask trunnions and demonstrate that they meet the guidance of NUREG-0612.

Criteria

NUREG-0612, *Control of Heavy Loads at Nuclear Power Plants* (Ref. A-19), Section 5.1.6(3) Single-Failure-Proof Handling Systems states:

Interfacing lift points such as lifting lugs or cask trunnions should also meet one of the following for heavy loads handled in the area where the crane is to be upgraded unless the effects of a drop of the particular load have been evaluated and shown to satisfy the evaluation criteria of Section 5.1:

(a) Provide redundancy or duality such that a single lift point failure will not result in uncontrolled lowering of the load; lift points should have a design safety factor with respect to ultimate strength of five (5) times the maximum combined concurrent static and dynamic load taking the single lift point failure.

OR

(b) A non-redundant or non-dual lift point system should have a design safety factor of ten (10) times the maximum combined concurrent static and dynamic load.

Peach Bottom Shipping Cask Information

The maximum package weight is conservatively assumed to be 65,000 (57,100 is the total cask weight without impact limiters + 7,900 is the largest payload weight for use at the ISF Facility) pounds. The trunnions have an outside diameter of 8 inches, and an inside diameter of 6 inches. The trunnion load is applied 2.5 inches from the outer shell of the cask.

The material properties are taken from the Westinghouse Safety Analysis Report (Ref. A-26) and are tabulated below:

Material: Type 304 stainless steel							
Components: Cavity Shell, Trunnions and Trunnion Plates, Outer Shell Overlay, End Plugs, Cask End Plates							
Property	70°F	100°F	200°F	300°F	400°F	500°F	Units
S ultimate	75	---	66	62.5	59.0	57.5	10 ³ psi
S yield	30	---	26	23.5	21.0	19.5	10 ³ psi
E	28.3	---	27.8	27.3	26.7		10 ⁶ psi
α mean	8.46	---	8.79	9.00	9.19		10 ⁻⁶ / F
ρ	0.290						lb/in ³
ν	0.3						
Where S ultimate = static ultimate strength (minimum), S yield = static yield strength (minimum), E = Young's Modulus of Elasticity, α mean = coefficient of thermal expansion, ρ = density, ν = Poisson's Ratio.							

Evaluation

There are two pairs of trunnions on the Peach Bottom cask. One pair is used with a special lifting device to rotate, lift and move the cask.

The lifting configuration is classified as a non-redundant or non-dual lift point configuration. Hence, the design safety factor of 10 on ultimate strength is applicable.

Static Load on Trunnion

$$W = 65,000 \text{ pounds (total weight of the package)}$$

$$P_s = \text{Static load to be supported on each trunnion}$$

$$P = 0.5W = 32,500 \text{ lb}$$

Dynamic Load on Trunnion

Dynamic load takes into account inertia forces and uncertainties in allowing for other influences. To account for dynamic influences a 15% increase in dead weight is assumed. The 15% factor is based on guidance from Crane Manufacturers Association of America (CMAA-70, Specification 70, *Specifications for Top Running Bridge and Gantry Type Multiple Girder Electric Overhead Traveling Cranes* (Reference A-27).

$$P_D = \text{Dynamic load to be supported on each trunnion}$$

$$P_D = 0.15P_s = 4,875 \text{ lb}$$

Load on Trunnion

The load on the trunnion is the sum of the static and dynamic load.

P = Trunnion Load

$$P = P_s + P_D = 37,375 \text{ lb}$$

Geometric Information for Trunnion

D_o = Outside diameter of trunnion = 8 inches

D_i = Inside diameter of trunnion = 6 inches

r_o = Trunnion outer radius = D_o/2 = 4 inches

I = Moment of inertia of trunnion

$$I = \frac{\pi(D_o^4 - D_i^4)}{64} = \frac{\pi(8^4 - 6^4)}{64} = 137.4 \text{ in}^4$$

A = Area of cross-section of trunnion

$$A = \frac{\pi(D_o^2 - D_i^2)}{4} = \frac{\pi(8^2 - 6^2)}{4} = 21.99 \text{ in}^2$$

L = Distance from P to junction of trunnion and shipping cask = 2.5 inches

M = Moment at junction = P x L = 37,375 lbs x 2.5 inches = 93,438 in-lbs

Maximum bending stress for trunnion

$$\sigma_B = \frac{M r_o}{I} = \frac{93,438 \text{ in} \cdot \text{lb} \times 4 \text{ in}}{137.4 \text{ in}^4} = 2,720 \text{ psi}$$

Maximum shear stress for trunnion

$$\tau = \frac{P}{A} = \frac{37,375 \text{ lb}}{21.99 \text{ in}^2} = 1,700 \text{ psi}$$

Maximum principal bending stress

The maximum trunnion principal stress (S_I) is determined by combining shear stress and bending stress as follows:

$$S_I = \frac{\sigma_B}{2} + \left[\left(\frac{\sigma_B}{2} \right)^2 + \tau^2 \right]^{0.5} = 3,537 \text{ psi}$$

Material properties

The maximum cask surface temperature at 100°F ambient temperature is predicted to be 161°F (with a 3850 Btu/hr internal heat load (Ref. A-26). For conservatism, use material properties of ultimate strength at 200°F.

$$S_u = 66,000 \text{ psi}$$

Design Safety Factors

The design safety factor, ϕ_u (ultimate), for the trunnions is:

$$\phi_u = S_u/S_1 = 18.6 > 10$$

Conclusions

The trunnions of the Peach Bottom shipping cask satisfy the design safety factors defined in NUREG-0612 for a non-redundant or non-dual lift point configuration for a single-failure proof handling system.

4.7.3.3.2 Coatings

The transfer casks, original top and bottom lids, and bolts are uncoated. The exposed surfaces of the transfer casks and original top and bottom lids are stainless steel and do not require any type of coating.

A second top lid, which is used in the transfer of Peach Bottom Core 1 fuel, is constructed from carbon steel and does require the use of a coating. The surface of the carbon steel lid (Figures A-21 and A-22) was prepared by glass beading and was coated with Keeler & Long white epoxy paint No. 3500, which is Kolor-Poxy Self-Priming Surfacing Enamel (Item 40 on Figure A-21). Based on the vendor's website information, the Keeler & Long No. 3500 product is one of a series of products that is classified as a protective coating system for nuclear power plants Service Level II, III, and balance of plant.

With respect to use in the fuel transfer to the ISF Facility, the lid will be exposed to atmospheric environments (air, rain, snow) and may be exposed to a potential temperature range of -30°F to 163°F. The -30°F corresponds to the lowest temperature during which DOE-ID plans to move fuel. The 163°F corresponds to the maximum ambient air temperature expected to occur in the transfer tunnel or fuel packaging area (refer to SAR Chapter 4 Table 4.2-50). The low-decay heat values of the fuels are such that they will not heat the lid up to 163°F.

The ISF Facility fuel transfers are a series of dry transfers versus any type of wet fuel pool transfers. As such, the coatings will be subjected to normal atmospheric environments. The Keeler & Long technical datasheet for the No. 3500 Kolor-Poxy Self-Priming Surfacing Enamel identifies the temperature resistance is 350°F. Hence, the coating is compatible with the range of potential environments.

4.7.3.4 Criticality Evaluation for Spent Fuel Handling Operations

Chapter 4 of the ISF Facility SAR presents criticality evaluations that address handling and storage operations after the Peach Bottom transfer cask has been opened in the Fuel Packaging Area (FPA).

The criticality analyses for the fuels when they are inside a closed Peach Bottom transfer cask are discussed in this Appendix.

Criticality analyses were performed (References A-13 and A-14) to confirm that the transfer configurations for each fuel type will result in a k_{eff} less than 0.95.

- A summary of the bounding criticality evaluations for each of the three fuel types in a Peach Bottom transfer cask is provided in Table A5.1-2.
- No critical configuration was identified for the Shippingport reflector modules or reflector loose rods that would be transferred in the Peach Bottom cask.
- For the Peach Bottom fuel the maximum number of elements in a given transfer cask is 18 (See Sections 4.7.1.2.1 and 4.7.1.2.2 of this Appendix). Since 18 is less than the number of elements identified in Table A5.1-2, no critical configuration is identified.
- For the TRIGA fuel, 90 elements are in a Peach Bottom transfer cask (See Section 4.7.1.2.4 of this Appendix.). The criticality analyses show that a change in the transfer packaging configuration is required in order to exceed a k_{eff} of 0.95.

For the TRIGA fuel, subcritical conditions are maintained by geometry and configuration control. TRIGA fuel geometry and configuration are provided by the 5-position standard TRIGA fuel can (Figure A-38) positioned within the TRIGA Bucket (Figure A-31). Three tiers of TRIGA Buckets are then positioned axially within the Lighter Weight Storage Canister Assembly (Figure A-27). A Gap Plug (Figure A-39) is then placed on top of the third TRIGA Bucket to fill the void in the Lighter Weight Storage Canister Assembly. The criticality analysis performed by DOE-ID (Reference A-13) modeled the stainless steel five-position standard TRIGA fuel can (Figure A-38) with 1.75-in diameter tubes. Credit is taken for the geometry of the five-position standard TRIGA fuel cans. The tubes that make up the cans are assumed to retain the fuel, and to remain centered on a circle of 2.977 in. minimum and 3.368 in. maximum diameter. No credit is taken for the stainless steel material that composes the tube walls of the fuel cans. The cans were modeled in an annulus, with the cans touching. This is closer than the actual configuration of cans in the TRIGA bucket, but is conservative. The aluminum TRIGA bucket is neglected except that the cans are positioned as if the bucket were present. Two separate drop analyses of the TRIGA fuel in the 5-position standard TRIGA fuel can (Reference A-3) have been performed. The first represents a 27-foot vertical drop with a bottom end orientation only. The second represents a 10-foot side drop. The conclusions of the analyses are that the welds joining the end fixtures to the standard aluminum and stainless steel TRIGA elements cannot be shown conclusively to remain intact during the two postulated drops. Regardless of weld performance, the fuel cylinders themselves will be protected by the graphite reflectors; will remain intact; and will remain confined to the individual tubes of the 5-position standard TRIGA fuel can. The 5-position standard TRIGA fuel cans will sustain only minor deformation.

For handling operations within the ISF Facility, the dropping of the Peach Bottom transfer cask is not considered to be a credible event (Refer to Sections 8.1.1.2 and 8.2.1.2 of this Appendix). Since no credible event has been identified that would result in a change to the geometry or configuration of the TRIGA fuel, subcritical conditions are maintained.

5.0 OPERATION SYSTEMS

5.1 OPERATION DESCRIPTION

The procedures in Section 5.1.1 of the ISF Facility SAR address the extent of cask operations within the ISF Facility. Further details regarding loading of the PB-1 and PB-2 casks at the INTEC dry fuel storage facilities have been provided in References A-9 through A-12.

5.1.1 Narrative Description

This topic is addressed in the ISF Facility SAR.

5.1.2 Flowsheets

This topic is addressed in the ISF Facility SAR.

5.1.3 Identification of Subjects for Safety Analysis

5.1.3.1 Criticality Prevention

This section summarizes the principal design features, procedures, and special techniques used to preclude criticality in the PB transfer cask.

An overview of this criticality prevention has been prepared by taking Table 3.3-5, *Control Methods for Prevention of Criticality* from the ISF Facility SAR and identifying principal design features, procedures, and special techniques associated with each control method. This overview is provided in Table A5.1-1, *Summary of Criticality Prevention*, of this Appendix. Control methods that are neither planned procedures nor principal design features have been classified as “special techniques” in this subsection.

Additional discussion of the criticality calculations and results can be found in the ISF Facility SAR Chapter 4 (Sections 4.2.3.3.7, 4.7.3.4, Appendix 4A) and Chapter 8.

Technical Criteria Associated with the Control Methods

The technical criteria associated with the control methods for prevention of criticality are summarized below:

No Mixing of Fuel Types. The criticality evaluations assume no mixing of the fuel types in the DOE-ID provided transfer cask (i.e., Peach Bottom, TRIGA, and Shippingport). For purposes of the following criticality discussion, the Peach Bottom Core 1 and Peach Bottom Core 2 fuels will be considered as one type even though their packaging configurations differ.

Number of Fuel Elements. Bounding criticality evaluations were performed for each of the fuel types to identify the minimum number of fuel elements and the spacing among the elements that is required to result in a configuration with k_{eff} greater than 0.95. Both dry and flooded conditions were evaluated in order to address normal, off-normal, and accident conditions. These cases are summarized in Table A5.1-2 of this Appendix. These criticality evaluations were performed (Reference A-13 and A-14) for INL onsite transfer of SNF using Peach Bottom casks.

Mass of Loose Material. The evaluations also considered SNF as loose material. Peach Bottom fuel is considered for this case since the cladding of the Shippingport reflector modules and TRIGA fuel elements would retain any loose material. In the dry condition, 36 crushed Peach Bottom fuel elements were needed to achieve a k_{eff} of 0.902.

Physical Separation of Sets of Fuel Elements. One method of criticality control is to ensure, through engineered physical separation, that fuel elements cannot be physically brought closer together into configurations that result in k_{eff} exceeding 0.95.

Geometric Control Provided by Canister and Packaging Structure. One method of criticality control is to ensure through control of fuel geometry with engineered features of the canister and packaging structures such that fuel elements cannot be physically arranged in configurations that result in k_{eff} exceeding 0.95.

Use of Burnup Credit. Not used.

Use of Burnable or Fixed Neutron Absorbers (Poisons). Not used.

5.1.3.1.1 Fuel in Transfer Cask

The principal design features, procedures, and special techniques that provide limitations on the amount of fissile materials and engineered safety features when the fuel is in the Transfer cask are provided below. Criticality analyses for the fuel configurations while in the Transfer Cask were performed (Reference A-13 and A-14).

No Mixing of Fuel Types

No mixing of fuel types is achieved through the following special techniques and procedures.

Special Technique. The Peach Bottom fuel will be shipped first. The Shippingport reflector modules will be shipped second. The TRIGA fuel will be shipped third. This shipping schedule sequence is the first step to prevent mixing of fuel types.

Procedure. A fuel manifest to be provided with each shipment. This document will be prepared under the ISFSI Quality Assurance Program (Ref A-24). Completion, checking, and approval of this manifest should serve to identify inadvertent mixing of fuel types. The manifest will be reviewed against license requirements and accepted prior to commencement of cask unloading operations.

Procedure. The fuel repackaging is planned as separate campaigns. The FPA is configured differently for each of the fuel types. When the transfer cask is opened, if the packaging is not consistent with the fuel type shown on the manifest, the fuel transfer operation will cease. The transfer cask will be bolted closed and returned for reuse.

Number of Fuel Elements

A control on the number of fuel elements in the transfer cask is achieved through the following special technique.

Special Technique. As stated in Section 3.1.1 of the ISF Facility SAR, the Shippingport reflector modules contained no fissile material at beginning of life. Further, as described in the ISF Facility SAR, Chapter 4, Appendix 4A, *Criticality Models*, the amount of enrichment that occurred during reactor operations results in this fuel containing insufficient fissile material to reach a k_{eff} of 0.95 or greater. Hence, there is no limit on the amount of Shippingport Reflector modules in the transfer cask with respect to maintaining subcriticality. The number of Peach Bottom and TRIGA fuel elements that can be placed in a transfer cask are limited by the design of the baskets and fuel containers that are loaded into the transfer cask. The fuel packaging for each fuel type is discussed in Section 4.7.1.2 of this Appendix.

Mass of Loose Fissile Material

Special Techniques. For the different fuel types, the separation of the uranium from the fuel matrix is considered unlikely based on the manufacturing processes associated with the respective fuel matrix. DOE-ID has performed analyses (Ref. A-14) to determine the mass of Peach Bottom Core 1 and Core 2 fuel required to achieve k_{eff} greater than 0.95. These analyses assumed the fuel elements were completely rubblized. Assuming dry conditions greater than 18 elements of Peach Bottom Core 1 fuel and greater than 12 elements of Peach Bottom Core 2 were required to exceed a k_{eff} of 0.95. Since the Peach Bottom Core 1 shipments are limited to 18 elements, and Peach Bottom Core 2 shipments will not exceed 12 elements, no special techniques are required. Shippingport reflectors and TRIGA elements must be intact as part of the acceptance criteria for preparing them for transfer. The cladding of the Shippingport reflectors and TRIGA fuel elements prevent the occurrence of loose fissile material.

Physical Separation of Sets of Fuel Elements by Engineered Features and Geometric Control Provided by DOE-ID Canister and Packaging Configurations

Special Technique. During the Shippingport repackaging campaign, criticality control is maintained by the radionuclide composition of the reflector. Therefore physical separations of sets of reflector elements and geometric control are not required.

Principal Design Features. The canister and packaging configurations for Peach Bottom and TRIGA fuels are required to provide the necessary geometry control and configuration to ensure that criticality safety requirements of the ISF Facility are met. Drop analyses on the Peach Bottom cask (Reference A-2) as well as the Peach Bottom cask containing Peach Bottom and TRIGA fuel containers (References A-3, A-5, and A-6) have been performed. The results of these analyses have shown that the containers provide physical separation of all sets of fuel elements and maintain geometric control. For the Shippingport reflector modules, there is insufficient fissile material for a criticality concern, therefore, no design features are required.

5.1.3.2 Chemical Safety

Not applicable to the Peach Bottom cask.

5.1.3.3 Operation Shutdown Mode

This topic is addressed in the ISF Facility SAR.

5.1.3.4 Instrumentation

The Peach Bottom casks are each fitted with four quick disconnect fittings. These are located within the Peach Bottom cask trunnions (see Figure A-5).

5.1.3.5 Maintenance Techniques

The Peach Bottom cask maintenance is controlled and administered through DOE-ID's support contractor. Routine inspections performed on the Peach Bottom cask are summarized in Section 4.5 of this Appendix.

5.2 FUEL HANDLING SYSTEM

This topic is addressed in the ISF Facility SAR.

5.3 OTHER OPERATING SYSTEMS

Not applicable to the Peach Bottom transfer cask.

5.4 OPERATION SUPPORT SYSTEMS

Not applicable to the Peach Bottom transfer cask.

5.5 CONTROL ROOM AND CONTROL AREAS

Not applicable to the Peach Bottom transfer cask.

5.6 ANALYTICAL SAMPLING

This topic is addressed the ISF Facility SAR.

6.0 SITE-GENERATED WASTE CONFINEMENT AND MANAGEMENT

The transfer of SNF to the ISF Facility in the Peach Bottom-1 and Peach Bottom-2 casks will not create any waste at the ISF Facility beyond those described in Chapter 6 of the ISF Facility SAR.

7.0 RADIATION PROTECTION

7.1 ENSURING THAT OCCUPATIONAL RADIATION EXPOSURES ARE ALARA

This topic is addressed in the ISF Facility SAR.

7.2 RADIATION SOURCES

This topic is addressed in the ISF Facility SAR.

7.3 RADIATION PROTECTION DESIGN FEATURES

7.3.1 Installation Design Features

This topic is addressed in the ISF Facility SAR.

7.3.2 Shielding

The Peach Bottom cask is used to transfer the fuel to the ISF Facility. The cask was originally designed to shield irradiated and unirradiated fuel such that the radiation levels on contact do not exceed 200 mrem/hr. Shielding analysis was performed, as documented in Section 7.3.2 of the ISF Facility SAR, to determine the dose rates for handling TRIGA fuel. The TRIGA source term was selected because it provides the bounding photon flux for all fuel types (see ISF Facility SAR Section 7.3.2). This configuration included the maximum 90 TRIGA elements that transfer in the cask. The results of the shielding analysis show that the expected surface dose rates are well within the Peach Bottom cask design criteria of 200 mrem/hr on contact. The peak combined dose rate at contact will be less than 33 mrem/hr.

7.3.3 Ventilation

Not applicable to the Peach Bottom transfer cask

7.3.4 Area Radiation and Airborne Radioactivity Monitoring Instrumentation

This topic is addressed in the ISF Facility SAR.

7.4 ESTIMATED ONSITE COLLECTIVE DOSE ASSESSMENT

This topic is addressed in the ISF Facility SAR. The calculated dose rates within the ISF Facility for the Peach Bottom cask is 11.2 mrem/hr at 1 foot for the bounding TRIGA fuel.

7.5 HEALTH PHYSICS PROGRAMS

This topic is addressed in the ISF Facility SAR.

7.6 ESTIMATED OFFSITE COLLECTIVE DOSE ASSESSMENT

This topic is addressed in the ISF Facility SAR. The ISF Facility controlled area boundary is 13,700m from the facility; therefore, dose rates at the offsite boundary are considered insignificant for normal transfer activities. Offsite dose calculations resulting from a cask drop accident have been performed that to result in only 0.02 mrem TEDE (Reference A-7).

8.0 ACCIDENT ANALYSES

Credible off-normal events and accidents within the ISF Facility are discussed in the ISF Facility SAR. Supporting analyses have been prepared to address bounding off-normal events and accidents specific to transfer operations associated with the Peach Bottom cask containing SNF. This chapter of the Appendix discusses off-normal and accident conditions involving the cask that could occur within the ISF Facility. The historical Peach Bottom cask safety analysis report (Ref A-16) for packaging and the analyses

performed (References A-2 through A-14) have been incorporated into the off-normal and credible cask-handling accidents that could occur at the ISF Facility discussed in the following sections.

8.1 CASK OFF-NORMAL OPERATIONS AT ISF FACILITY

The following off-normal events are postulated and evaluated for the Peach Bottom casks at the ISF Facility. These off-normal scenarios and their respective evaluations have been compared with the analyses (References A-2 through A-14) to confirm that the analyses are bounding for credible and postulated off-normal events at the ISF Facility involving the Peach Bottom cask.

8.1.1 Transfer Cask Events

8.1.1.1 Cask Pressurization

Misventing of a pressurized cask is evaluated in Section 8.1 of the ISF Facility SAR. Additional information on the impact of pressurization on the Peach Bottom casks follows.

Before transfer to the ISF Facility, the transfer cask is loaded under atmospheric conditions. It is unlikely that the transfer cask is capable of retaining an internal pressure since the O-rings will not be installed for fuel transfers to the ISF Facility.

Pressure rise in the transfer cask during movement from the INTEC to the ISF Facility would result primarily from heating of the transfer cask airspace from internal and external heat sources. A breach during transfer of a fuel container could contribute to a limited additional pressure rise from the gas used to backfill the container. However, due to the short transfer distance and container inspection before transfer, failure is not expected. Due to the mass of the transfer cask and the relatively short transportation time, significant pressurization of the transfer cask is not expected under normal conditions.

To determine a bounding off-normal cask pressure, the cask is assumed to be loaded at the off-normal temperature of -40°F at atmospheric pressure. The Safety Analysis Report submitted with the application for a Certificate of Compliance for the Peach Bottom Cask (Reference A-16) performed a thermal evaluation of the cask assuming 130°F ambient air temperature, full solar insolation, and an internal heat load of 14, 250 BTU/hr. Under these conditions, the average internal temperature of the cask was calculated to be approximately 315°F. These conditions are more extreme than will be encountered for the ISF Facility transfers since the internal heat load is approximately an order of magnitude less than that used in the application for a COC, and the off-normal ambient air temperature is only 101°F. Using the ideal gas laws, the internal transfer cask expansion from increasing the temperature from -40°F to 315°F was estimated, to conservatively bound any realistic conditions for this configuration. The internal volume of the transfer cask for this temperature change is conservatively assumed to remain unchanged. Using the ideal gas law:

$$(P_1V_1)/T_1 = (P_2V_2)/T_2; \quad \text{or} \quad P_2 = ((T_2V_1)/(T_1V_2)) \times P_1$$

Where: $P_1 = 14.7$ psia or 1 atm; $V_1 = V_2 =$ Units cancel; $T_1 = -40^\circ\text{F}$ or 233°K ; $T_2 = 315^\circ\text{F}$ or 430.2°K

$$P_2 = (430.2/233) \times 1.0 \text{ atm} = 1.85 \text{ atm or } 12.5 \text{ psig}$$

As discussed above this calculation is very conservative. Even under this very conservative evaluation the pressure increase is significantly less than the 100 psig rating of the cask.

This event involves no change to the fuel or structural integrity configuration. Hence, there is no change in criticality, confinement, or retrievability of the spent nuclear fuel.

8.1.1.2 Cask Drop Less Than Design Allowable Height

This event is evaluated in Section 8.1 of the ISF Facility SAR. Additional information on the impact of a drop on the Peach Bottom casks follows.

Dropping the Transfer Cask during handling is not considered a credible off-normal event. The cask receipt crane and interfacing lifting devices are designed in accordance with the guidance contained in NUREG-0612 (Ref. A-19). The cask receipt crane is designed in accordance with NUREG-0554, *Single-Failure-Proof Cranes for Nuclear Power Plants* (Ref. A-18). The cask lifting yoke that is used to lift the cask from the transporter and loading into the Cask Trolley is designed as specified in ANSI N14.6-1993 (Ref. A-20), with the more conservative design margins specified in NUREG-0612 applied. The Peach Bottom cask trunnions are evaluated in Section 4.7.3.3.1 of this Appendix and are also demonstrated to meet the single failure proof design margins specified in NUREG-0612. Therefore, dropping the transfer cask during hoisting operations is not considered a credible event.

8.1.2 Fuel Packaging Events

8.1.2.1 Attempt to Lower Fuel Container Into Occupied Fuel Station

The postulated cause, detection, and analysis of this event are provided in the ISF Facility SAR. The analysis identified a maximum impact deceleration force that may be applied to the fuel containers, transported in the Peach Bottom cask. The containers were designed specifically for handling and storing these fuels. ISF Facility operations associated with these containers will not introduce any unusual handling hazards, and the containers are expected to maintain integrity and prevent damage to the fuel during these off-normal events.

8.1.3 Fuel Storage Events

Not applicable to the Peach Bottom transfer cask

8.1.4 Waste Handling Events

Not applicable to the Peach Bottom transfer cask.

8.1.5 Other Events

8.1.5.1 Ventilation System Failures

Not applicable to the Peach Bottom transfer cask.

8.1.5.2 Loss of External Power Supply for a Limited Duration

Not applicable to the Peach Bottom transfer cask.

8.1.5.3 Off-Normal Ambient Temperatures

This event is evaluated in Section 8.1 of the ISF Facility SAR. Additional information on the impact of off-normal ambient temperatures on the Peach Bottom casks follows.

The Peach Bottom casks are designed to withstand environmental extremes, including extended off-normal ambient temperatures, without adverse effects to the fuel. The ISF cask-handling operations are bounded by the DOE-ID transportation requirements and evaluations. The thermal analysis presented in *Safety Analysis for the Shipment of Peach Bottom No. 1 Irradiated Fuel Elements in Whitehead & Kales Shipping Cask Model No. PB-1* (Ref. A-16) assumed a cask heat load of 14,250 Btu/hr decay heat plus 3540 Btu/hr insolation. Under normal conditions of transport with ambient air temperature between -40°F and 130°F were assumed for the analysis. To maximize the cask temperature extremes, the cask was assumed to be exposed to direct sunlight when ambient air temperature was 130°F , and shade was assumed when temperature was -40°F . These temperatures bound the off-normal external air temperature range of the ISF Facility (-40°F to 101°F). The calculated maximum temperatures in the Peach Bottom-1 cask were based on the highest ambient temperature (130°F with solar insolation):

hottest fuel canister temperature:	450°F
hottest basket temperature:	375°F
cask outer shell temperature:	240°F
cask outer overlay shell temperature:	200°F

The Peach Bottom-1 cask analysis assumes a cask is loaded with 19 Peach Bottom elements, each with a maximum decay heat of 750 Btu/hr. The maximum total heat load in the cask is then determined by multiplying the number of fuel elements by the maximum per element decay heat rate. The maximum cask heat load from the SNF is 14,250Btu/hr (4176 watts). The Peach Bottom 1 basket structure is fabricated from aluminum tubes and plates.

This can be considered a bounding thermal analysis for the Peach Bottom-1 cask when used to transfer Peach Bottom 1 and 2 fuel, TRIGA aluminum or stainless clad fuel, and Shippingport reflector modules.

The thermal load of the Peach Bottom 1 cask when delivering ISF fuel elements is shown in Table A8.1-1.

The highest incoming heat load in the Peach Bottom 1 cask is 12 Peach Bottom Core 2 elements. At approximately 40 watts, this is two orders of magnitude less than the maximum design heat load for the Peach Bottom-1 cask. Therefore the ISF fuel heat loads will not challenge the thermal and structural performance of the Peach Bottom-1 cask.

8.1.6 Radiological Impact From Off-Normal Operations

This topic is addressed in Section 8.1 and Table 8.1-1 of the ISF Facility SAR.

8.2 ACCIDENTS

The following accidents are postulated and evaluated for the Peach Bottom casks at the ISF Facility.
Transfer Cask Events

8.2.1.1 Vehicular Collision With Transporter

Cause of Accident

Collision of a vehicle with a Peach Bottom cask during transfer from the ISF site boundary fence to the Cask Receipt Area is postulated to occur as a result of human error. Impact of the transporter with the ISF Facility structure is addressed in Section 8.2 of the ISF Facility SAR. Impact between the cask transporter and another vehicle is discussed below.

Accident Analysis

As discussed in Chapter 2 of the ISF Facility SAR, the ISF Facility is on a controlled site where vehicle access is restricted. The closest public highway with uncontrolled access is approximately 10 miles from the ISF. Roads closer to the ISF are access controlled. Therefore, collisions with vehicles originating from offsite are not postulated to occur.

Onsite ISF traffic is minimal, and is administratively controlled during spent fuel transfer activities. However, a vehicular collision with the Peach Bottom transfer cask transport vehicle could be postulated. Any postulated accident would occur between the site fence and the facility entrance and would be limited with respect to speed, frequency, and consequences by the short distance and the site control procedures. The potential frequency and consequences would be conservatively bounded by postulated transportation accidents during transit to the ISF site. The Peach Bottom casks were previously licensed for over-the-road highway transport under COC No. USA/6375/B()F (Ref A-23) and, as such, were demonstrated to withstand postulated transportation accidents. The analyzed configuration included the use of the cask impact limiters. The safe transportation of the Peach Bottom transfer casks without installed impact limiters has been demonstrated in the analyses (Reference A-2).

Radiological Consequences

The PB Cask drop evaluation discussed in Section 3.3.2.1.1 of this Appendix bound the radiological consequences from this postulated event.

8.2.1.2 Transfer Cask Drop Scenarios

8.2.1.2.1 Transfer Cask Drop During Hoisting Operations

This accident is evaluated in Section 8.2 of the ISF Facility SAR. The evaluation of the cask trunnions to NUREG-0612 criterion was presented in Section 4.7.3.3.1 of this Appendix. No additional analysis was performed for this postulated ISF Facility cask drop since the lifting configuration described in the ISF Facility SAR ensures that a drop is not credible, as summarized in Section 8.1.1.2 of this Appendix.

8.2.1.2.2 Transfer Cask Drop Off the Transport Trailer

The transport trailers for the transfer casks (Trailer No. 71801 and 71808) were not classified as important to safety since they do not provide functions associated with confinement, criticality control, heat removal, or protection of the fuel containers. In addition, no credible failure of a transport trailer has been identified that would prevent the transfer cask from performing its important to safety functions during spent fuel transfer to the ISF Facility.

As part of the original safety analysis prepared in support of the original COC (Ref A-16), analyses of a 30-foot drop onto a hard-unyielding surface in end, side, and corner orientations were provided in order to demonstrate the cask's performance in meeting 10 CFR Part 71 acceptance criteria. This analysis included consideration of the dampening effect from impact limiters.

An updated analysis for a Peach Bottom cask drop without impact limiters (Reference A-2). The purpose of the analysis was to evaluate the capability of the cask structure to maintain a confinement barrier for the enclosed contents during the drop scenarios considered. The drop analysis was performed for two different cask configurations. The first configuration is Peach Bottom Cask configured to transfer the Peach Bottom Core 2 fuels, Shippingport reflector modules, and TRIGA fuels. The cask configurations for each of these are shown on Figures A-2 through A-4, respectively. The second Peach Bottom cask configuration is used to transfer the Peach Bottom Core 1 fuel, which is shown on Figure A-1. The worst-case cask loadings were used for each of these configurations. For the first configuration, a Shippingport package weight of 10,000 pounds was assumed. For the second configuration a 3260 pound loaded weight of the basket/canister assembly was used. Three drop scenarios were selected as bounding for the planned operations and shipping configurations. These scenarios are:

- a) A 27 foot drop using configuration 1 described above. This analysis was based on installation of the original lead-filled lids retained with eight bolts in each lid.
- b) An 8 foot drop using configuration 1 described above. This analysis was based on installation of the original lead-filled lids retained with eight bolts in the top lid and four bolts installed in the bottom lid.
- c) An 8 foot drop using configurations 2 described above. This analysis was based on installation of a solid carbon steel top lid with eight bolts installed and the original lead-filled bottom lid with only four bolts installed.

These above analyses were performed based on impact on an unyielding surface. An analysis associated with a 10 foot drop onto a concrete surface was also performed. The results of these analyses concluded that cask confinement barrier capability was maintained.

8.2.1.3 Transfer Cask Tipover

This topic is addressed in the ISF Facility SAR.

8.2.1.4 Cask Trolley Collision Events

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of a cask trolley collision on the Peach Bottom cask follows.

The cask trolley is used to move the loaded cask into the ISF Facility's Transfer Tunnel and to return the empty cask to the Cask Receipt Area of the ISF Facility. The cask trolley rides on rails. This limited movement path prevents any credible impacts on the structural members or fixed location components. However, the ISF Facility's inner and outer Transfer Tunnel doors and canister trolley operate on the same track, and therefore are potential impact sites during movement of the cask trolley. Proximity sensors in the Transfer Tunnel provide positional information for control and operational interlock functions and initiation of speed controller functions (deceleration, etc.) to facilitate accurate positioning at each trolley stopping station. Over travel of the trolley is prevented by end-of-travel shunt limit switches hardwired into the drive contactor control circuit. In addition, bumpers at each end minimize impact against the canister trolley, rail-mounted end stops, or inadvertently closed doors. The cask trolley is designed for impacts at velocities up to the maximum operating speed of 10 fpm without damage to the trolley or cask. The cask trolley impact against the rail stops or canister trolley will be minimized by bumpers designed for an average rate of deceleration of 4.7 ft/sec^2 (0.15 g) in accordance with CMAA 70 (Ref. A-27). The resulting stresses on the cask from this deceleration are well within the bounds of a 3-foot side drop previously analyzed in the Safety Analysis for Shipment of LWBR Fuel in the Peach Bottom BP-2 Cask from ECF to ICPP (Ref A-26). That analysis resulted in a "g" loading on the cask of 41.5 g. The analysis demonstrated that the trunnion would partially crush but the cask outer shell would not puncture, the impact would not result in a permanent bend in the shell, and the shell welds would not yield. Thus a 0.15 g force that would result from a Cask Trolley collision event will cause no significant damage to the cask.

8.2.2 Fuel Packaging Events

8.2.2.1 Failure of DOE-ID Fuel Container During Handling

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of a fuel container failure follows.

This accident consists of dropping a fuel container from the maximum height of the ISF Facility's fuel handling machine (FHM) into the Peach Bottom cask, or onto the FPA floor. The loaded fuel containers are handled by single-failure-proof lifting devices as described in Section 4.7 of the ISF Facility SAR. The structural integrity of the fuel container attachment points to the suspension system, and capability to support the postulated loads have been analyzed and shown to be acceptable for loading into the Peach Bottom cask and transfer to the ISF Facility.

The consequences of a drop event involving the fuel containers have been analyzed. At the ISF Facility, lifting of these fuel containers only occurs from the Transfer Cask into the FPA or within the FPA. A drop of a fuel container would not create a criticality concern as discussed in Section 4.7.3.4 of this Appendix. The drop could result in damage to the fuel and the fuel container, but the release of radioactive material would occur within the confinement barrier of the FPA, thereby minimizing the release to the environment. Section 8.2.4.5 of the ISF Facility SAR provides a discussion of the Maximum Hypothetical Accident Dose.

8.2.2.2 Drop of ISF Basket During Handling

Not applicable to the Peach Bottom transfer cask.

8.2.2.3 Canister Trolley Movement in Raised Position

Not applicable to the Peach Bottom transfer cask.

8.2.3 Fuel Storage Accidents

Not applicable to the Peach Bottom transfer cask.

8.2.4 Other Postulated Accidents

8.2.4.1 Adiabatic Heatup

Not applicable to the Peach Bottom transfer cask.

8.2.4.2 Loss of Shielding

Postulated events that could impact shielding on the transfer cask include vehicular collision with the transporter, transfer cask drop or tipover, fire and explosion, or tornado missile impact, all of which are covered in subsections of 8.2 of this Appendix.

8.2.4.3 Building Structural Failure onto Structures, Systems, or Components

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of building structural failure on the Peach Bottom casks follows.

Failure of the building structures is not considered credible due to natural phenomena or overstressing of lifting mechanisms. However, the truck used for transporting the Peach Bottom casks containing SNF could hit the Cask Receipt Area structure. ISF procedures limit the use and speed of vehicles within the controlled area to minimize the potential for vehicle impact. A cask drop could result from an impact to the structure with sufficient force to cause structure damage. The only scenario with a cask in the elevated position, and with the potential for sufficient vehicle speed (and therefore sufficient impact energy), would occur while loading the empty cask on the truck for return. As there would be no SNF involved, there would be no significant radiological consequences. A transporter impact event involving a loaded cask could only occur while the cask was on the truck and the truck was being moved into the facility. In this event, cask drop would be minimized and the fuel container would be protected by the transfer cask. The scenario would be bounded by the analyzed cask drop accident (Reference A-2). Once unloading of the cask begins, the truck moves slowly and in limited increments within the Cask Receipt Area.

8.2.4.4 Fire and Explosion

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of fire and explosion on the Peach Bottom casks follows.

For fire hazard evaluation purposes, the ISF Facility is divided into three fire areas, each of which is divided into multiple fire zones. Fire Area 1, which includes the Transfer Tunnel through which the cask passes, and Fire Zone 1 of Fire Area 3, which includes the Cask Receipt Area, are of primary concern for cask operations. Fire and explosion hazards for the ISF Facility are discussed in depth in Section 8.2.4.4 of the ISF Facility SAR.

The worst-case fire loading in the areas of the ISF Facility where the Peach Bottom cask is handled amounts to less than a 30-minute equivalent fire duration. This is not a safety concern for the Peach Bottom casks, as the original Peach Bottom cask safety analysis (Ref A-16) for the Peach Bottom fuels demonstrated that the cask met the regulatory criteria of 10 CFR 71 for the hypothetical fire accident. After exposure to a 1475°F fire for 30 minutes, the maximum temperature of the lead shielding and inner liner of the cask was 426°F. Since lead melts at 621°F, no lead melt occurs and there is no loss of shielding integrity. Because the maximum fire duration will be less than 30 minutes based on combustible loading, the cask inner liner and fuel temperatures will be less than 426°F.

8.2.5 External Events

8.2.5.1 Loss of External Power for an Extended Interval

Not applicable to the Peach Bottom transfer cask.

8.2.5.2 Earthquake

This topic is addressed in the ISF Facility SAR.

8.2.5.3 Flood

This topic is addressed in the ISF Facility SAR.

8.2.5.4 Extreme Wind

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of a tornado on the Peach Bottom casks follows.

The design basis tornado is analyzed in Section 8.2 of the ISF Facility SAR. Two cases defined and analyzed therein involve performance of the Peach Bottom casks to maintain a confinement barrier for the SNF.

Case 1 - Outside Receipt Area

While a Peach Bottom cask is inside the ISF Facility site boundary but outside of the Cask Receipt Area, it is subject to design basis tornado (DBT) winds, missiles and differential pressures. The Peach Bottom cask has been analyzed to survive drops of up to 27 feet without loss of the cask confinement barrier. This analysis is considered to bound the cases of overturning the cask/trailer by the DBT winds and the DBT missile loadings.

Case 2 - Inside Receipt Area, Peach Bottom Cask on Transporter, Unsecured with Impact Limiters Removed

Case 2 assumes a Peach Bottom cask is on the transporter, but is within the Cask Receipt Area with the cask tiedown attachments removed. As in Case 1, the confinement barrier is provided by the cask. DBT loadings were utilized in the design of the Cask Receipt Area structure, therefore, this structure is assumed not to fail and impact the transfer cask during this event. The impact of non-structural members of the Cask Receipt Area (e.g., the sheet metal siding) is bounded by the Spectrum II missiles assumed in the analysis in the ISF

Facility SAR, Chapter 8. Therefore, analyses of the DBT loadings remain limited to the loadings identified in Table 3.2-1 of the ISF Facility SAR. The evaluations discussed above bound this configuration as well.

8.2.5.5 Lightning

This accident is evaluated in Section 8.2 of the ISF Facility SAR. Additional information on the impact of lightning on the Peach Bottom casks follows.

The Peach Bottom casks will be briefly transiting from the ISF Facility gates to the covered Cask Receipt Area. The casks will be resting on rubber-tired transporters during the time that they are outside the facility. The rubber tires will insulate the cask and transporter from ground, thereby preventing a significant surge of electrical current from passing through the cask body in the event of a lightning strike.

8.2.5.6 Accidents at Nearby Sites

This topic is addressed in the ISF Facility SAR.

8.2.5.7 Volcanism – Basaltic Lava Flow

This topic is addressed in the ISF Facility SAR.

8.2.5.8 Aircraft Impact

This topic is addressed in the ISF Facility SAR.

8.3 SITE CHARACTERISTICS AFFECTING SAFETY ANALYSIS

This topic is addressed in the ISF Facility SAR.

9.0 CONDUCT OF OPERATIONS

Conduct of operations at the ISF Facility is addressed in the ISF Facility SAR. Conduct of operations specific to loading of the Peach Bottom casks at fuel storage facilities on the INL site and transfer to the ISF Facility site is under the DOE-ID regulatory jurisdiction. An overview of the operations associated with the fuel transfer process is provided in Figures A-1 through A-4.

10.0 OPERATING CONTROLS AND LIMITS

Operating controls and limits within the ISF Facility are addressed in the ISF Facility SAR.

11.0 QUALITY ASSURANCE

Quality assurance relative to the ISF Facility, and operations therein, is the responsibility of DOE-ID under the ISFSI Quality Assurance Program (Ref A-24). Quality assurance associated with the Peach Bottom transfer casks, and the SNF to be transferred therein, is also the responsibility of DOE-ID. All work performed in support of this contract by DOE-ID and/or its support contractor is being performed under the ISFSI Quality Assurance Program.

With respect to the SNF, a shipping manifest will be provided for each fuel shipment. The manifest will be provided to for review as part of the onsite acceptance of the shipment. This manifest will be prepared by DOE-ID under the established ISFSI Quality Assurance Program (Ref A-24).

12.0 REFERENCES

- A-1. U.S. Department of Energy Idaho Operations Contract No. DE-AC07-00ID13729, *Spent Nuclear Fuel Dry Storage Project*, <http://www.id.doe.gov/doiid/psd/SNFDSPContract.htm>.
- A-2. U.S. Department of Energy Idaho Operations, Engineering Design File 3151, *Drop Evaluation of the Peach Bottom Cask Without External Impact Limiters*, Rev 2, March 3, 2003.
- A-3. U.S. Department of Energy Idaho Operations, Engineering Design File 2251, *TRIGA Fuel Damage and Tier Mixing in Postulated SNFDSP Shipment Accidents*, Rev 1, July 3, 2002.
- A-4. U.S. Department of Energy Idaho Operations, Engineering Design File 2280, *Analysis of LWBR Storage Liner Drops*, Rev 0, June 12, 2002.
- A-5. U.S. Department of Energy Idaho Operations, Engineering Design File 2233, *Drop Evaluation of the Peach Bottom Cask Overpack in the Peach Bottom Cask*, Rev 0, June 5, 2002.
- A-6. U.S. Department of Energy Idaho Operations, Engineering Design File 2281, *Peach Bottom Canister in Cask Ten Foot Drop Evaluation Rev 0*, June 12, 2002.
- A-7. U.S. Department of Energy Idaho Operations, Engineering Design File 1552, *Dose Calculations for Spent Fuel Drop Accidents*, Rev 0, June 10, 2002.
- A-8. U.S. Department of Energy Idaho Operations, Engineering Design File 4190, *Supplementary Drop Evaluation of the Peach Bottom Casks*, Rev 1, October 23, 2003.
- A-9. U.S. Department of Energy Idaho Operations, Engineering Design File 2873, *Peach Bottom Core 1 Shipping Configuration for Transfer from CPP-749 to Spent Nuclear Fuel Dry Storage Project (Ref Dwg. 518304)*, Rev 2, June 19, 2002.
- A-10. U.S. Department of Energy Idaho Operations, Engineering Design File 2874, *Peach Bottom Core 2 Shipping Configuration for Transfer from CPP-603 Irradiated Fuel Storage Facility (IFSF) to Spent Nuclear Fuel Dry Storage Project (Ref Dwg 518306)*, Rev 2, June 20, 2002.
- A-11. U.S. Department of Energy Idaho Operations, Engineering Design File 2875, *Shippingport LWBR Spent Nuclear Fuel Data for Transfer from CPP-749 to Spent Nuclear Fuel Dry Storage Project (Ref Dwg 518305)*, Rev 1, June 20, 2002.
- A-12. U.S. Department of Energy Idaho Operations, Engineering Design File 2876, *TRIGA Spent Nuclear Fuel Data for Transfer from IFSF to Spent Nuclear Fuel Dry Storage Project (Ref Dwg 518307)*, Rev 1, June 20, 2002.
- A-13. U.S. Department of Energy Idaho Operations, INEEL/INT-02-00294, *Criticality Safety Evaluation of TRIGA and LWBR Fuels in the Peach Bottom Cask*, Rev 1, June 2002.
- A-14. U.S. Department of Energy Idaho Operations, INEEL/EXT-02-00008, *Criticality Safety Evaluation: Peach Bottom Unit 1 Fuel in the Peach Bottom Casks and in the Underground Fuel Storage Facility*, March 2002.
- A-15. Title 10, Code of Federal Regulations, Part 72, *Licensing Requirements for Independent Storage of Spent Nuclear Fuel and High-Level Radioactive Waste*.

- A-16. *Safety Analysis for the Shipment of Peach Bottom No. 1 Irradiated Fuel Elements in Whitehead & Kales Shipping Cask Model No. PB-1*, January 15, 1970, Battelle Memorial Institute, Columbus, Ohio.
- A-17. *Proposed Decommissioning Plan*, ISF-FW-PLN-0027, Docket No. 72-25, Foster Wheeler Environmental Corporation, Richland, Washington
- A-18. *Single Failure-Proof Cranes for Nuclear Power Plants*, NUREG-0554, U.S. Nuclear Regulatory Commission, Washington, D.C., May 1979
- A-19. *Control of Heavy Loads at Nuclear Power Plants, Resolution of Generic Technical Activity A-36*, NUREG-0612, U.S. Nuclear Regulatory Commission, Washington, D.C., July 1980
- A-20. ANSI (1993), *Special Lifting Devices for Shipping Containers Weighing 10,000 Pounds (4500 kg) or More*, ANSI N14.6, American National Standard Institute, Washington, D.C.
- A-21. Wenzel, D.R. and B.J. Schrader, April 2001, *The Radiological Safety Analysis Computer Program (RSAC-6) User's Manual*, INEEL/EXT-01-00540.
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- A-23. Certificate of Compliance, Peach Bottom 1 Cask, COC USA/6375/B()F, Docket No. 71-6375, Nuclear Regulatory Commission, Washington, D.C.
- A-24. DOE/RW-0333P, Revision 10, ISFSI Quality Assurance Program
- A-25. DOE (1997), *Foreign Research Reactor Fuel Acceptance Criteria Failed Fuel Report*, DOE/SNF/REP-013, November 1997, U.S. Department of Energy, National Spent Nuclear Fuel Program, Idaho Falls, Idaho.
- A-26. Westinghouse (1986), *Safety Analysis for Shipment of LWBR Fuel in the Peach Bottom PB-2 Cask from ECF to ICPP*, WAPD-LP(CE)FD-50, Bettis Atomic Power Laboratory, Westinghouse Electric Corporation, West Mifflin, Pennsylvania.
- A-27. Crane Manufacturers Association of America, Specification No. 70, Revised 1994, *Specifications for Top Running Bridge and Gantry Type Multiple Girder Electric Overhead Traveling Cranes*.

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Appendix A Tables

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**Table A3.3-1
Transfer Cask Key Equipment Selected to Provide Protection to the Spent Nuclear Fuel**

Equipment Name	Key Equipment Items	Key Design Criteria
Transfer Cask – Peach Bottom casks PB-1 and PB-2	Cask	10 CFR 71 (January 1970) (per Ref. A-16) 3715 Watt internal heat source term (per Ref. A-16) Welding procedures for cask construction “are essentially those of the ASME Boilers and Pressure Vessel Code Section VIII Unfired Pressure Vessels, 1962 Edition” (per Ref. A-16)
	Trunnions	10 CFR 71.31 (January 1970) (support 3 times the loaded weight) (per Ref. A-16)

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Table A4.7-1
Peach Bottom Cask Dimensions and Weights⁽¹⁾⁽²⁾

Overall length with lids	170 inches
Overall length with impact limiters	191 inches
Cask cavity length	159 inches
Outside diameter	42.6 inches
Cask cavity inside diameter	26 inches
Trunnion span	50 inches
Trunnion spacing (center-to-center)	12 ft-10 inches ⁽²⁾
Trunnion outside diameter	8 inches
Trunnion length	4 inches
Cask lid thickness	7 inches (original configuration) 6.25 inches (new A36 steel top lid)
Cavity wall inner thickness	0.25 inches
Cask outer wall thickness	1.5 inches
Outer skin (cladding) thickness	0.25 inches
Lead shielding thickness	6.25 inches in central 110-inch section of cask length; 5.25 inches for 24.5-inch length from bottom of cavity at each end 4.0 inches at each end lid (original configuration)
Impact limiter length (ea.)	9 inches
Lid bolt diameter	1.25 inches
Cask body weight	53,110 lb
Cask lid weight (ea.)	1,995 lb (original lead-filled design) ~1,841 lb (new A36 steel top lid)
Total cask weight (empty)	58,260 lb (original with impact limiters) 57,100 lb (original w/o impact limiters) ~56,946 lb (with new A36 steel top lid and w/o impact limiters)

Notes:

- (1) Dimensions from Peach Bottom cask design drawings in Figures A-5 through A-7 and from Reference A-16.
- (2) PB-1 and PB-2 actual dimensions have been field-verified and are shown on Figures A-32 through A-34).

Table A4.7-2
Peach Bottom Cask Construction Materials

Cask body	Carbon steel, AISI 1025 (original material)
Cask body outer skin	Stainless steel, Type 304 (replacement material)
Inner cavity shell	Stainless steel, Type 304 (original material)
Trunnions	Stainless steel, Type 304 (original material)
Trunnion reinforcing plates	Stainless steel, Type 304 (original material)
Cask Top Lid #1 ⁽¹⁾	6.25" thick, carbon steel, Type A36 (INEEL design) (new material)
Cask Top Lid #2 ⁽²⁾	4" lead sandwiched between two plates of 1.5" Type 304 stainless steel (original materials)
Cask Bottom Lid	4" lead sandwiched between two plates of 1.5" Type 304 stainless steel (original materials)
Cask closure lid bolts	ASTM A-276, UNS 21800 (Nitronic 60) (new material)

Notes:

- (1) Top Lid #1 to be used only with Peach Bottom Core 1 fuel canisters.
- (2) Top Lid #2 to be used for all other fuel transfers.

**Table A4.7-3
Transfer Cask Materials Mechanical Properties**

Material Property	Unit	Temperature °F					
		70	100	200	300	400	500
Cavity shell, Trunnions and Trunnion Plates, End Plugs, Cask End Plates (stainless steel portion of the original ss/lead/ss sandwich lids) (original materials); Outer Shell Overlay (replacement material): Type 304 Stainless Steel							
S _u	10 ³ psi	75	--	66	62.5	59.0	57.5
S _y	10 ³ psi	30	--	26	23.5	21.0	19.5
E	10 ⁶ psi	28.3	--	27.8	27.3	26.7	26.1
α _m	10 ⁻⁶ /°F	8.46	--	8.79	9.00	9.19	9.37
ρ	lb/in ³	0.290					
v		0.3					
Cask Outer Shell (original material): ASTM A36 (AISI 1025 Carbon Steel used for design properties)							
S _u	10 ³ psi	55	--	52.25	--	51.7	--
S _y	10 ³ psi	36	--	34.2	--	32.7	--
E	10 ⁶ psi	29.0	--	28.13	--	27.55	--
α _m	10 ⁻⁶ /°F	6.3	--	6.3	--	--	--
ρ	lb/in ³	0.283					
v		0.3					
Top Lid (new material): ASTM A36 Carbon Steel							
S _y	10 ³ psi	36					
Lid bolts (new material): ASTM A-276, UNS 21800 (Nitronic 60)							
S _u	10 ³ psi	95.0					
S _y	10 ³ psi	50.0					
Gamma shielding (original material): Cast Lead							
S _u	10 ³ psi	5.0					
S _y	10 ³ psi	1.0					
E	10 ⁶ psi	2.0					
ρ	lb/in ³	0.41					
Type 304 Stainless Steel Base Metals & Electrode Materials as Specified in 748-WP-101							
Base Metal (original materials)			Electrode Materials (original materials)				
Type	Material	S _u Ksi	Material				S _u Ksi
(1) Plate	ASTM A-167 TP 304	75.0	ASTM A-298 E-308-15 lime coated				80.0
(2) Pipe	ASTM A-312 TP 304	75.0					
(3) Bars	ASTM A-276 TP 304	75.0	ASTM A-371 GR-308 (TIG Process)				80.0
(4) Plate	ASTM A-240 TP 304	75.0	ASTM A-276-65 E-308-L				80.0
Tungsten electrodes are non-consumable and do not become part of a welded joint.			ASTM B-297 EWTh-2 (tungsten)				N A

Table A4.7-4 **Sheet 1 of 2**
Exceptions to Codes and Standards for the Transfer Casks

Reference Code or Standard Section/Article	Code or Standard Requirement	Exception, Justification & Compensatory Measures
ASME Section VIII 1962 Edition, Subsection A General Requirements UG-5 General.	(a) Material subject to stress due to pressure shall conform to one of the specifications in Section II of the Code and shall be limited to those that are permitted in the applicable Part of Subsection C, except as otherwise permitted in Paragraphs UG-10 and UG-11.	ASTM A-167 is not listed in the 1962 Edition of ASME Section VIII or Section II. ASTM A-167 had a 1963 and 1969 edition. A review of the Chemical Requirements (Table 1) and Mechanical Property Requirements (Table 2) for Type 304 in the 1969 Edition shows that they are the same as for ASTM A-240 TP304 in the 1962 Edition of Section VIII. In addition, the drawings have no specific call out for ASTM A-167. Addendum 4 (dated August 14, 1974) of the Safety Analysis Report provides actual materials stress properties. No further compensatory measures are considered necessary.
ASME Section VIII 1962 Edition, Subsection A General Requirements UG-5 General.	(a) Material subject to stress due to pressure shall conform to one of the specifications in Section II of the Code and shall be limited to those that are permitted in the applicable Part of Subsection C, except as otherwise permitted in Paragraphs UG-10 and UG-11.	ASTM A-276 is not listed in the 1962 Edition of ASME Section VIII or Section II. ASTM A-276 had 1962, 1963, and 1965 editions. This standard is for bar stock shapes. The only call outs for bar stock are on drawings BMI Dwg. Nos. 0029 Rev C, 0031 Rev C, and 0037 Rev A. The bar stock does not function as part of the pressure boundary or a load path. No further compensatory measures are considered necessary.
ASME Section VIII 1962 Edition, Subsection A General Requirements UG-5 General.	(a) Material subject to stress due to pressure shall conform to one of the specifications in Section II of the Code and shall be limited to those that are permitted in the applicable Part of Subsection C, except as otherwise permitted in Paragraphs UG-10 and UG-11.	ASME A-371 GR-308 (TIG Process) is not listed in the 1962 Edition of ASME Section II electrode material. ASME Section II 1962 Edition contains a specification for ASTM A-371. It lists ER-308. It is assumed that the GR-308 is the same as ER-308. No further compensatory measures are considered necessary.

Table A4.7-4 **Sheet 2 of 2**
Exceptions to Codes and Standards for the Transfer Casks

Reference Code or Standard Section/Article	Code or Standard Requirement	Exception, Justification & Compensatory Measures
ASME Section VIII 1962 Edition, Subsection A General Requirements UG-5 General.	(a) Material subject to stress due to pressure shall conform to one of the specifications in Section II of the Code and shall be limited to those that are permitted in the applicable Part of Subsection C, except as otherwise permitted in Paragraphs UG-10 and UG-11.	ASTM A-276-65 E-308-L is not listed in the 1962 Edition of ASME Section II electrode material. As noted above, ASME Section II 1962 Edition does not contain ASTM A-276. However, E-308-L is a standard electrode used with other ASME approved materials. No further compensatory measures are considered necessary.

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**Table A5.1-1
Summary of Criticality Prevention for Transfer Cask**

Control Methods for Prevention of Criticality1	Fuel in Transfer Cask
Limitation on the amount of Fissile Materials	
No mixing of fuel types	ST - Shipping schedule P - Fuel Shipment Manifest, Separate fuel repackaging campaigns PDF – Configurations of Canisters and Packaging
Number of fuel elements	ST- Shippingport reflector module composition PDF- Configurations of ISF Baskets
Mass of loose fissile material	ST – Fuel manufacturing process
Engineered Safety Features	
Physical separation of sets of fuel elements by engineered features	ST- Shippingport reflector module composition PDF – Canister and packaging structural design
Geometric control provided by	ST- Shippingport reflector module composition PDF – Canister and packaging structural design
Use of burnup credit	Not used
Use of burnable or fixed neutron absorbers (poisons)	Not used

Notes:

ST – Special Technique

P – Procedure

PDF – Principal Design Feature

**Table A5.1-2
Bounding Criticality Evaluations**

Type of Fuel	Dry as packed			Flooded		
	No. of elements	Optimum Spacing	k_{eff}	No. of elements	Optimum Spacing	k_{eff}
Shippingport Reflector Modules for Type IV and V ^a	Infinite array of pellets	In contact with each other	0.19	Infinite array of rods	1 inch	0.65
TRIGA Fuel Elements in Peach Bottom Cask ^b	65	In contact with each other on single tier of Cask	0.95	40	0.25 cm	0.95
Peach Bottom Core 1 & 2 Fuel Elements in Peach Bottom Cask ^c	37	In contact with each other	0.65	19	In contact with each other	0.94

Notes:

1. The FWENC analysis based for flooded condition bounds the DOE-ID evaluations
2. Bounding case to determine number of TRIGA elements on a single tier to exceed k_{eff} of 0.95
3. Bounding case for loose, bare elements in a tight-diameter packing. Flooded case assumes saturated graphite. (Greater than 37 fuel elements were not considered since the quantity is more than twice the largest existing INL fuel storage container.

**Table A8.1-1
Peach Bottom Cask Heat Load**

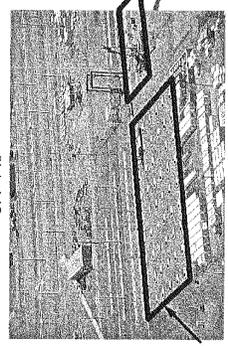
Fuel Type	Quantity in Peach Bottom Cask	Fuel Decay heat	Cask Heat Load
Peach Bottom 1	18 elements	0.053 watts per element	0.954 watts
Peach Bottom 2	12 elements	3.276 watts per element	39.312 watts
TRIGA	90 elements	0.326 watts per element	29.34 watts
Shippingport Reflector Modules IV	1 module	9.809 watts per module	9.809 watts
Shippingport Reflector Modules V	1 module	7.142 watts per module	7.142 watts
Shippingport Reflector Rods	127 rods	0.043 watts per rod	5.461 watts

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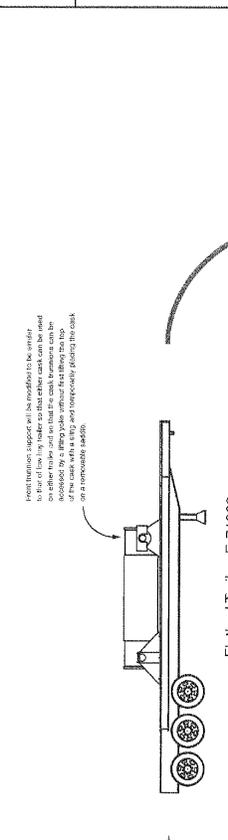
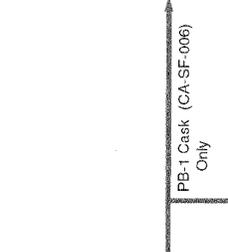
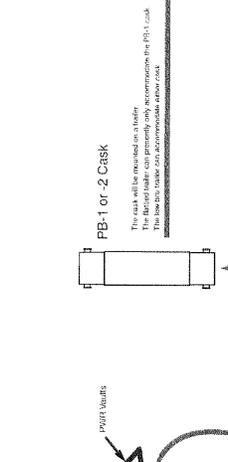
Appendix A Figures

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Interface Control Diagram Peach Bottom Core 1 Fuel Transfer from CPP-749



CPP-749

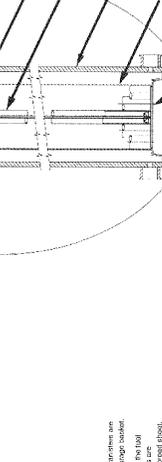
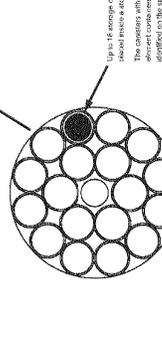
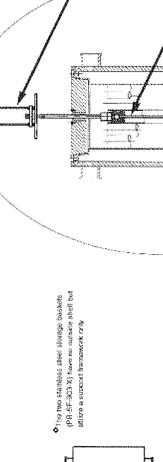
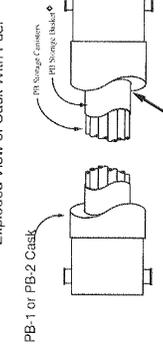
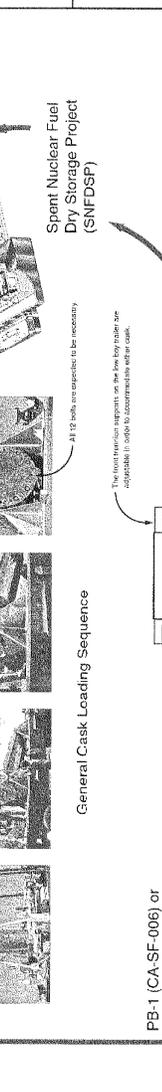
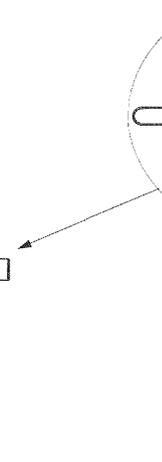


The Peach Bottom Core 1 fuel is stored in the fuel storage baskets (PB-1 and PB-2) at the CPP-749 facility. Each basket contains only one storage basket.

The Peach Bottom Core 1 fuel is currently stored in the CPP-749 facility in the fuel storage baskets (PB-1 and PB-2) at the CPP-749 facility. Each basket contains only one storage basket.

The PB-1 or -2 Cask is used to transport fuel from the CPP-749 facility to the SNF DSP facility. The PB Storage Basket is used to store fuel from the PB-1 or -2 Cask.

The Flatbed Trailer E-71808 is used to transport the PB-1 or -2 Casks. The Low Boy Trailer E-71801 is used to transport the PB Storage Baskets.



Steel Reinforced PB Storage Basket (BS-SF-807A) Drawings

Sheet No.	Rev. No.
09504	1
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09506	1

PB-1 or PB-2 Cask Drawings

Sheet No.	Rev. No.
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09730	1
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Flatbed Trailer Drawings

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Low Boy Trailer Drawings

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58898	1
58899	1
58900	1

The Peach Bottom Core 1 fuel is stored in the fuel storage baskets (PB-1 and PB-2) at the CPP-749 facility. Each basket contains only one storage basket.

The PB-1 or -2 Cask is used to transport fuel from the CPP-749 facility to the SNF DSP facility. The PB Storage Basket is used to store fuel from the PB-1 or -2 Cask.

The Flatbed Trailer E-71808 is used to transport the PB-1 or -2 Casks. The Low Boy Trailer E-71801 is used to transport the PB Storage Baskets.

The Spent Nuclear Fuel Dry Storage Project (SNF DSP) facility is used to store spent nuclear fuel.

Aluminum PB Storage Basket Drawings

Sheet No.	Rev. No.
50514	1
50515	1
50516	1
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50600	1

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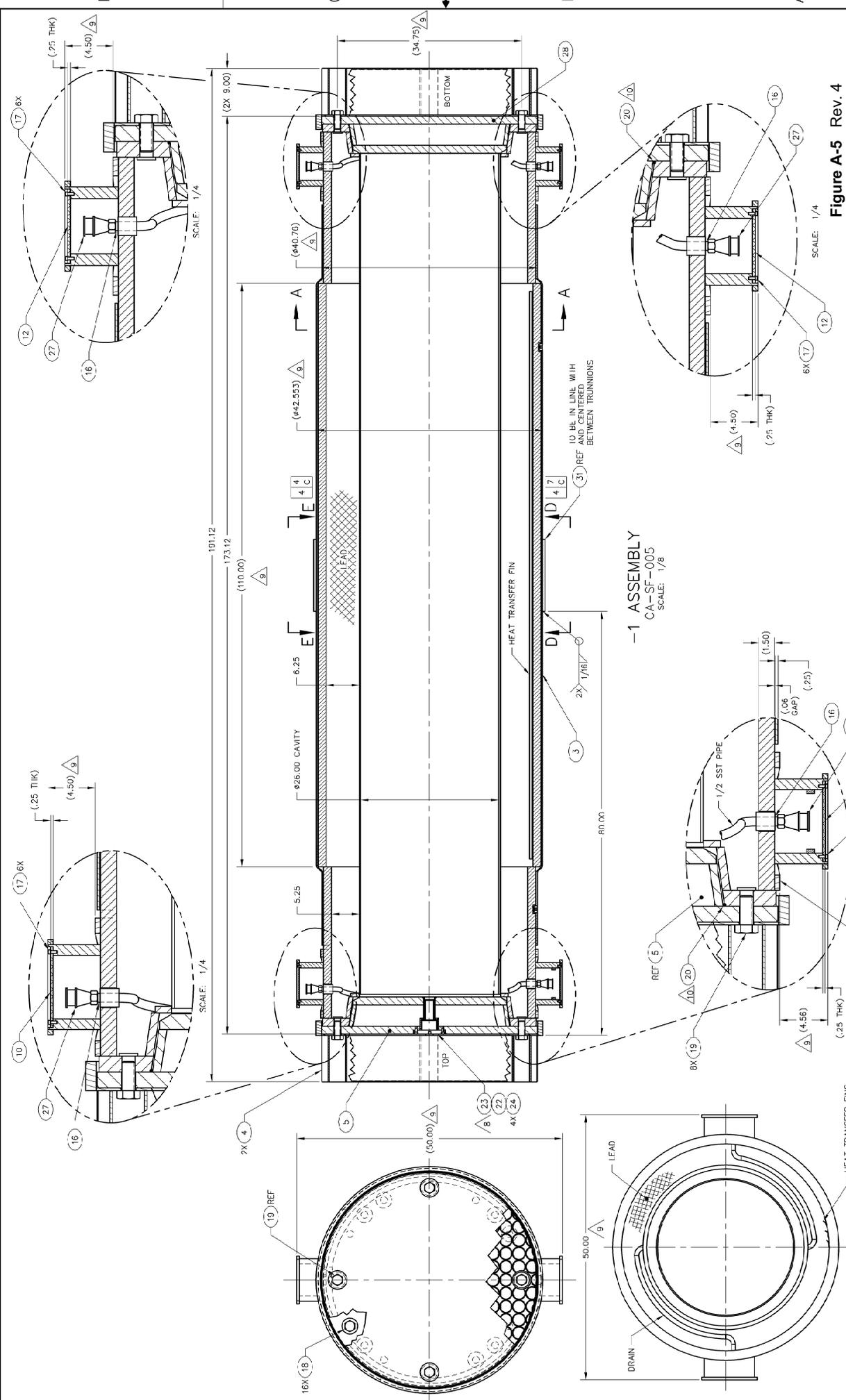
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The Spent Nuclear Fuel Dry

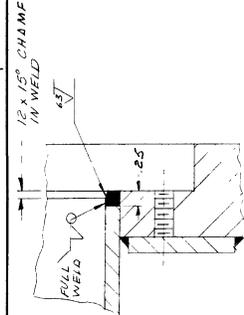


NOTES:

- DO FINAL MACH. AFTER WELDING.
- SEE DWG 0001 FOR WELD SPECS.
- PLUG CAVITY, COVER, ITEM #7 TO BE IN ONE COVER ONLY (TOP).
- AS-BUILT MEASUREMENT FOR THIS DIMENSION CAN BE FOUND ON DRAWING 509725.
- MARK PER STD-7006-2A IN 3/8 INCH HIGH CHARACTERS WITH INFORMATION GIVEN IN TABLE. COV-SF-XXX-X IS A UNIQUE IDENTIFICATION MARKING WHICH SHALL BE FILLED WITH BLACK INK PER STD-7008-12. MARKING SHALL CONTAIN LESS THAN 50 PPM.

CASK COVER	UNIQUE NUMBER
CA-SF-005	UPPER COV-SF-005-1
CA-SF-005	LOWER COV-SF-005-2
CA-SF-006	UPPER COV-SF-006-1
CA-SF-006	LOWER COV-SF-006-2

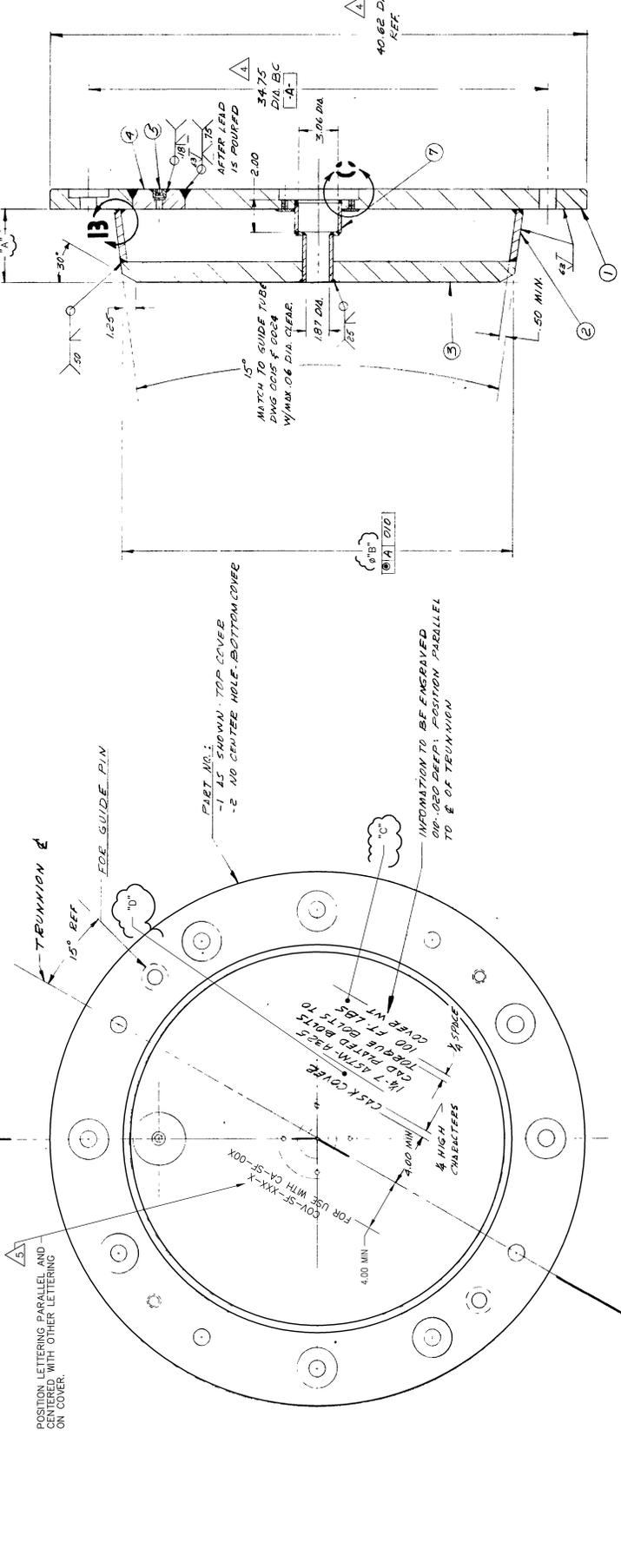
6. THIS IS A SAFETY CATEGORY, SAFETY SIGNIFICANT (SS) ASSEMBLY.



DETAIL B
SCALE: FULL

SAFETY CAT.	INTEL DRAWING NO.	REV.	DATE	DESCRIPTION
SS	500220	1	0007 D	TOP PLATE COVER
	500221	2	0008 D	SOFT COVER
	500222	3	0009 C	BOTTOM PLATE COVER
	200223	4	0010 B	PLUG COVER
		5		SAFETY FLAG; PATENTED ITEM FURNISHED BY THE EDWARD LEAD CO., COLUMBUS OHIO
		6		LEAD PLATE COVER
	200260	7	0001 B	PLUG CAVITY COVER

DETAIL C
(ONE COVER ONLY)
SCALE: FULL



SECTION A-A

Figure A-6 Rev. 4



TABULATION BLOCK A			
DIM	NOMINAL DIM	COV-SF-005-1	COV-SF-005-2
"A"	5.50	5.5	5.5
"B"	29.50	29.703	29.730
"C"	---	1900	1841
"D"	---	PB-1 (1972)	PB-1 (1972)

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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3	07/19/03	083	000	011	
4	07/19/03	083	000	011	

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2	04/24/03	083	000	011	
3	07/19/03	083	000	011	
4	07/19/03	083	000	011	

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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

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4	07/19/03	083	000	011	

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REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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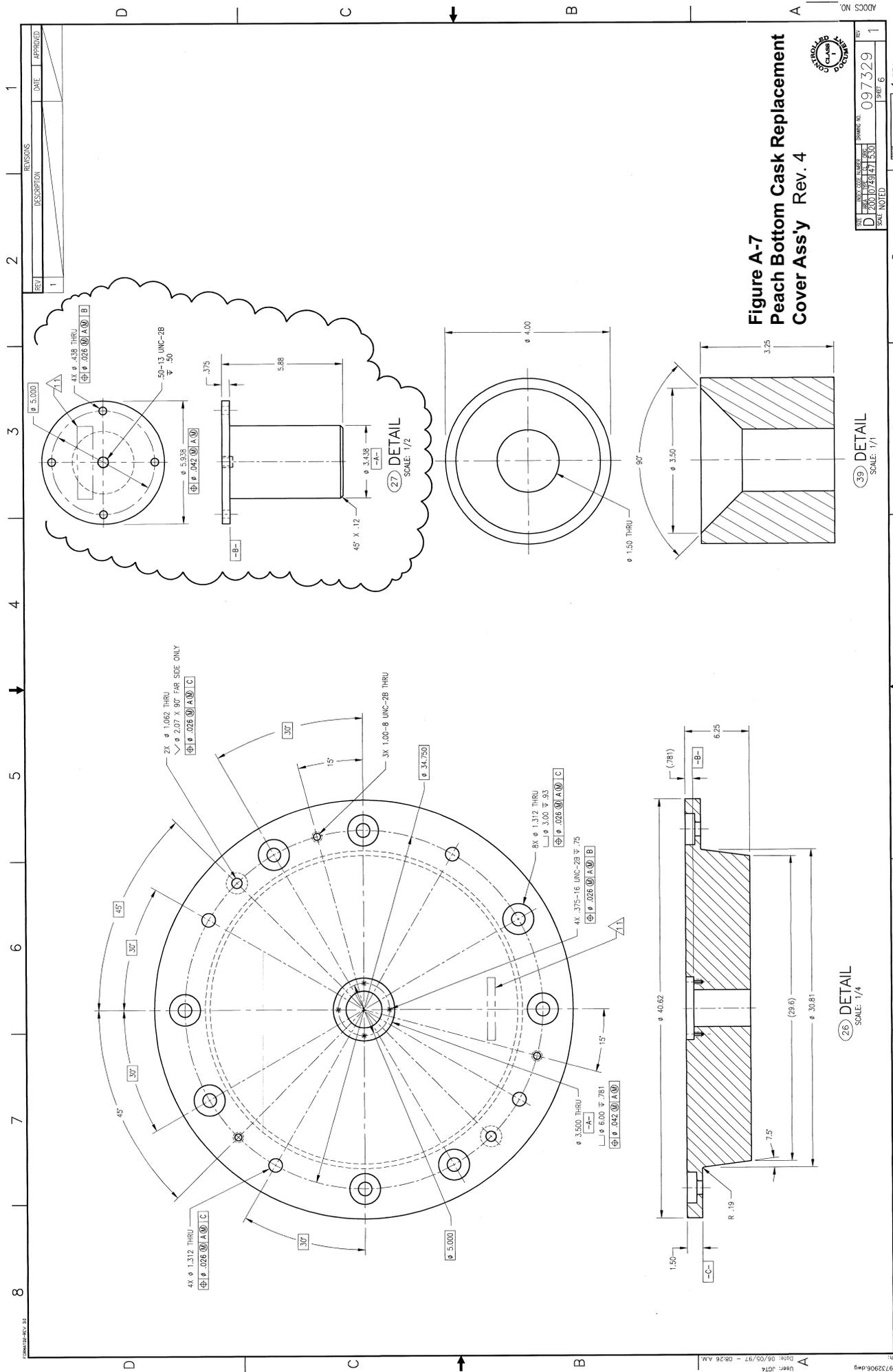
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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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4	07/19/03	083	000	011	

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4	07/19/03	083	000	011	

REV.	DATE	BY	CHKD.	APP'D.	DESCRIPTION
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2	04/24/03	083	000	011	



**Figure A-7
Peach Bottom Cask Replacement
Cover Assy Rev. 4**



REV	DESCRIPTION	DATE	APPROVED
1			

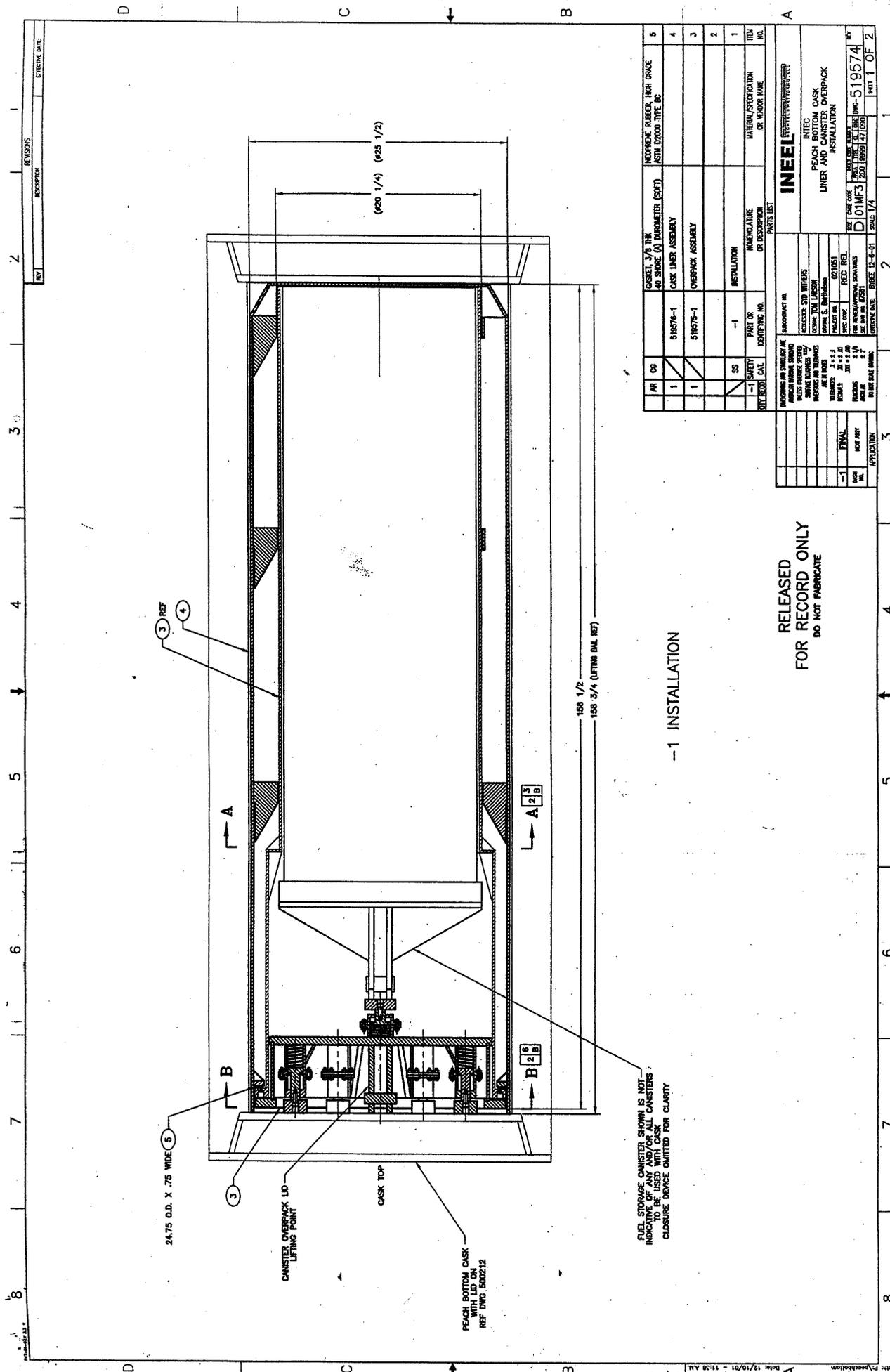
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SCALE	NOTED	SHEET	TBA
		6	1

25) DETAIL
SCALE: 1/4

26) DETAIL
SCALE: 1/4

27) DETAIL
SCALE: 1/2

28) DETAIL
SCALE: 1/1



REV	DESCRIPTION	DATE
1	ISSUED FOR CONSTRUCTION	12/10/71
2	REVISED TO REFLECT CHANGES TO THE CASK	1/15/72

NO.	DESCRIPTION	QTY	UNIT
1	INSTALLATION	1	FTU
2	OVERPACK ASSEMBLY	3	EA
3	CASK LINER ASSEMBLY	1	EA
4	GASKET 3/4" THK NEOPRENE RUBBER, HIGH GRADE 40 SHORE (A) DIMENSION (SOFT) ASTM D2000 TYPE BC	1	EA
5	PEACH BOTTOM CASK WITH LID ON REF. DWG 506212	1	EA

REV	DESCRIPTION	DATE
1	ISSUED FOR CONSTRUCTION	12/10/71
2	REVISED TO REFLECT CHANGES TO THE CASK	1/15/72

-1 INSTALLATION

RELEASED FOR RECORD ONLY
DO NOT FABRICATE

CASK STORAGE CANISTERS SHOWN IS NOT INDICATIVE OF ANY AND/OR ALL CANISTERS TO BE USED WITH CASK. CLOSURE DEVICE OMITTED FOR CLARITY.

INEEL (INTEGRATED NEUTRON ECONOMIZER)

PROJECT: PEACH BOTTOM CASK LINER AND CANISTER OVERPACK INSTALLATION

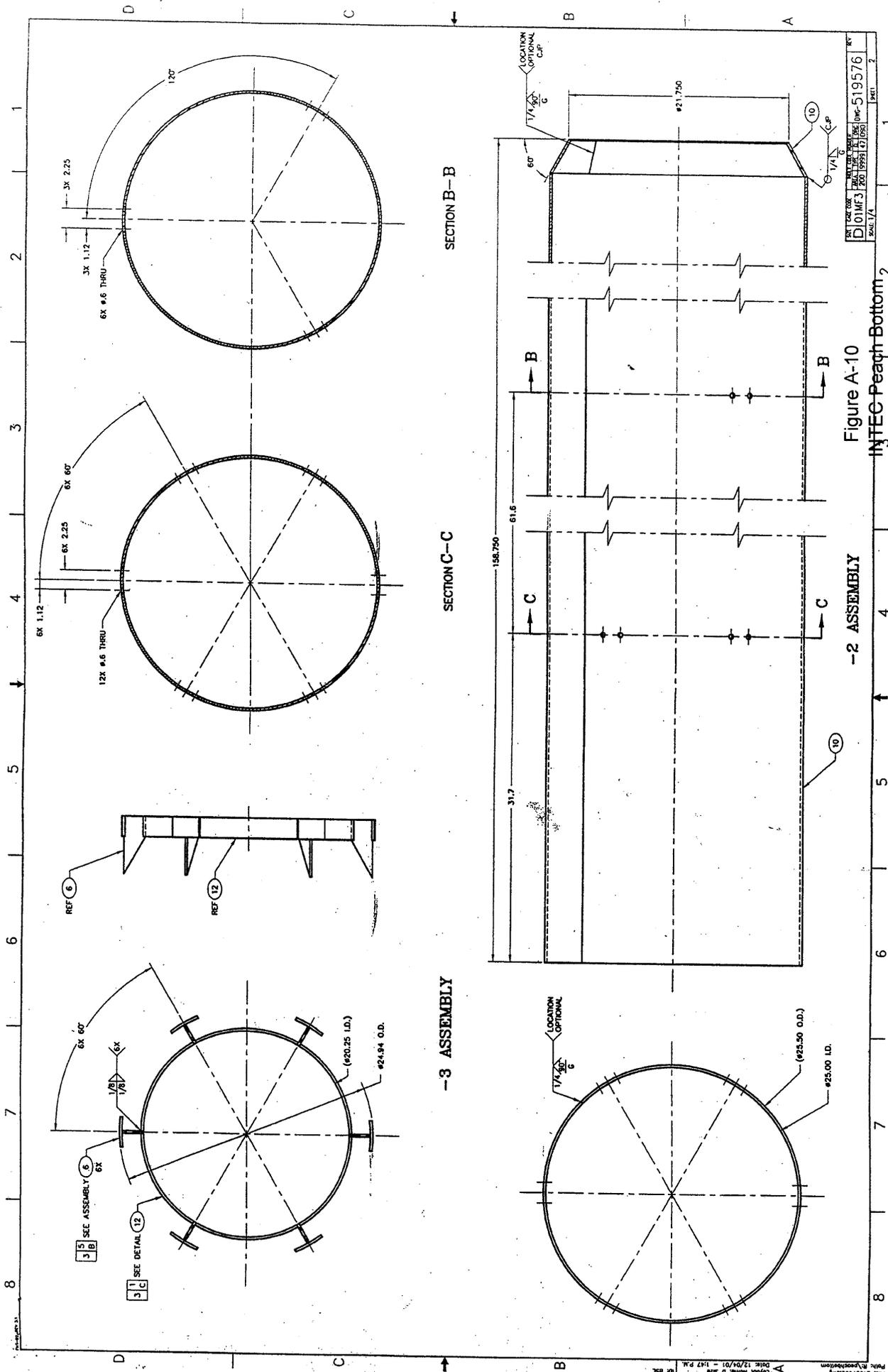
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BY: [Signature]

SCALE: 1/4" = 1'-0"

SHEET: 1 OF 2

Figure A-8 Rev. 4

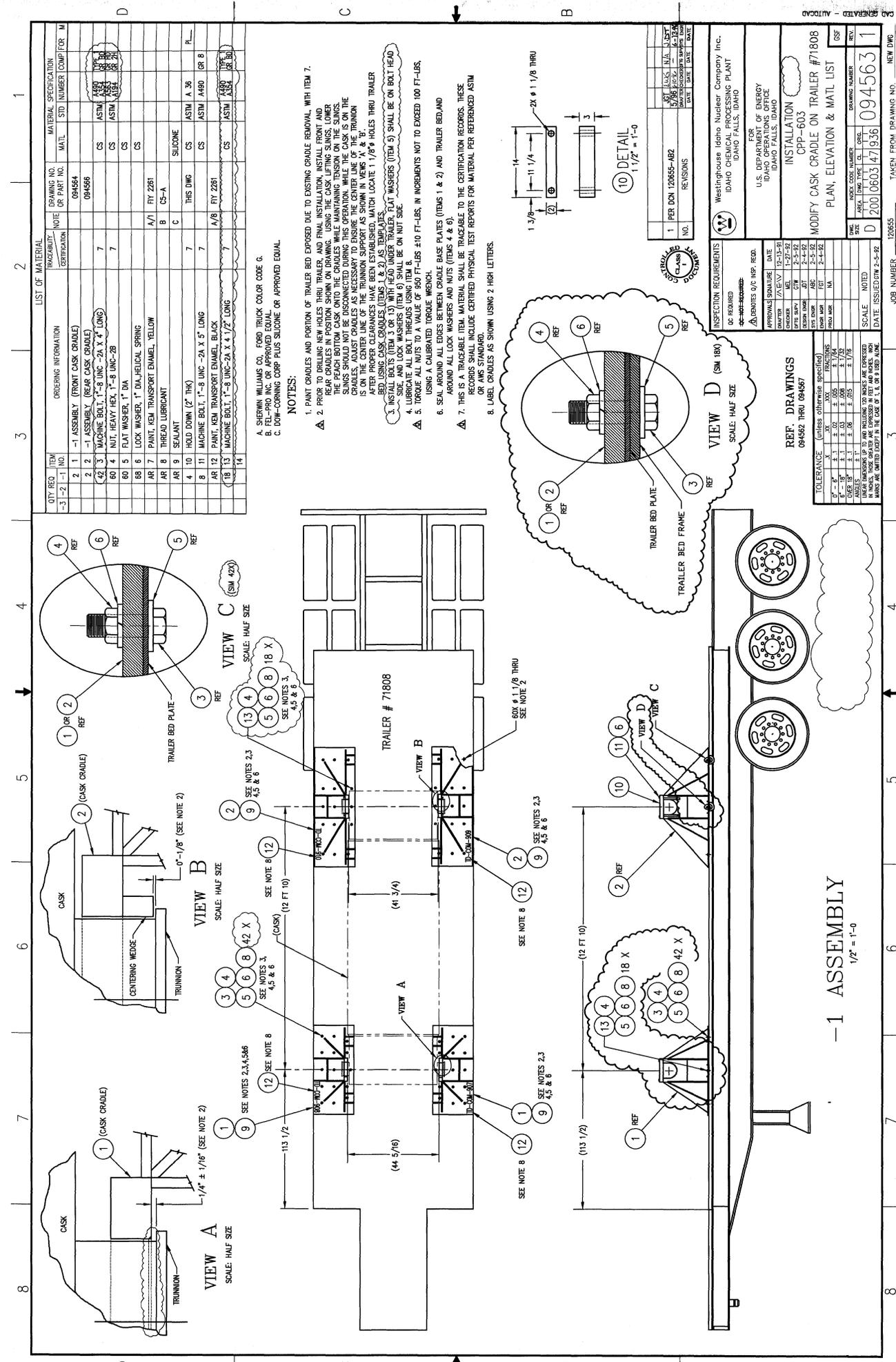


PK	DATE	BY	CHKD	DATE	BY
D	01/11/73	3	1	01/11/73	1
DWG-519576					
SCALE: 1/4"					
					SHEET 2

Figure A-10
INTEC Peach Bottom 2
Cask Assembly and Details

Rev. 4

File: 519576.dwg
Date: 12/29/01 1:47 PM
User: j...



LIST OF MATERIAL

QTY	ITEM	NO.	ORDERING INFORMATION	BACKSPLIT/ CERTIFICATION	NOTE OR PART NO.	DRAWING NO. OR PART NO.	MATERIAL SPECIFICATION
3	1	1	ASSEMBLY (FRONT CASK CRADLE)			094564	MATL STD NUMBER (COMP FOR) M
2	2	1	ASSEMBLY (REAR CASK CRADLE)			094566	
42	3	1	MACHINE BOLT, 1"-8 UNC - 2A X 4" (LONG)	7			CS ASTM A490 (OR 304)
60	4	1	NUT, HEAVY HEX, 1"-8 UNC - 2B	7			CS ASTM A490 (OR 304)
60	5	1	FLAT WASHER, 1" DIA				CS ASTM A490 (OR 304)
88	6	1	LOCK WASHER, 1" DIA, HELICAL SPRING				CS ASTM A490 (OR 304)
AR	7	1	PAIN, KEM TRANSPORT ENAMEL, YELLOW		A/1 RY 2281		
AR	8	1	THREAD LUBRICANT		B CS-A		
4	10	1	HOLD DOWN (2" THK)	7			SILICONE CS ASTM A 36
8	11	1	MACHINE BOLT, 1"-8 UNC - 2A X 5" LONG				CS ASTM A490 GR 8
AR	12	1	PAIN, KEM TRANSPORT ENAMEL, BLACK		A/8 RY 2281		
18	13	1	MACHINE BOLT, 1"-8 UNC - 2A X 1 1/2" LONG	7			CS ASTM A490 (OR 304)

NOTES:

1. PAIN CRADLES AND PORTION OF TRAILER BED EXPOSED DUE TO EXISTING CRADLE REMOVAL, WITH ITEM 7.
2. PRIOR TO DRILLING NEW HOLES THRU TRAILER, AND FINAL INSTALLATION, INSTALL FRONT AND REAR CASK CRADLES ON TRAILER BED. DRILL HOLES THROUGH CRADLES AND TRAILER BED. DRILL HOLES THROUGH EACH BOTTOM CASK ONTO THE CRADLES, WHILE MAINTAINING TENSION ON THE SLINGS. SLINGS SHOULD NOT BE DISCONNECTED DURING THIS OPERATION. WHILE THE CASK IS ON THE CRADLES, ADJUST CRADLES AS NECESSARY TO ENSURE THE CENTER LINE OF THE TRUNNION IS ON THE CENTER LINE OF THE TRUNNION SUPPORT AS SHOWN IN VIEWS 'A' & 'B'.
3. INSTALL CASK CRADLES (ITEMS 1 & 2) AS TEMP PLATES. THE CENTER LINE OF THE TRUNNION SUPPORT LOCATE 1 1/8" HOLES THRU TRAILER BED USING CASK CRADLES (ITEMS 1 & 2) AS TEMP PLATES.
4. INSTALL BALLS (ITEM 9 OR 13) WITH HEAD UNDER TRAILER. FLAT WASHERS (ITEM 5) SHALL BE ON BOLT HEAD SIDE, AND LOCK WASHERS (ITEM 6) SHALL BE ON NUT SIDE.
5. TORQUE ALL NUTS TO A VALUE OF 950 FT-LBS ±10 FT-LBS, IN INCREMENTS NOT TO EXCEED 100 FT-LBS, USING A CALIBRATED TORQUE WRENCH.
6. SEAL AROUND ALL EDGES BETWEEN CRADLE BASE PLATES (ITEMS 1 & 2) AND TRAILER BED AND AROUND ALL LOCK WASHERS AND NUTS (ITEMS 4 & 6).
7. THIS IS A TRACEABLE ITEM. MATERIAL SHALL BE TRACEABLE TO THE CERTIFICATION RECORDS. THESE RECORDS SHOULD INCLUDE CERTIFIED PHYSICAL TEST REPORTS FOR MATERIAL PER REFERENCED ASTM OR FEDERAL SPECIFICATION.
8. LABEL CRADLES AS SHOWN USING 2" HIGH LETTERS.

A. SHERWIN WILLIAMS CO. FORD TRUCK COLOR CODE G
 B. FEL-PRO INC. OR APPROVED EQUAL.
 C. DOWN-CORNING CORP PLUS SILICONE OR APPROVED EQUAL.

INSPECTION REQUIREMENTS

QC REQUIRED: YES NO

APPROVALS:

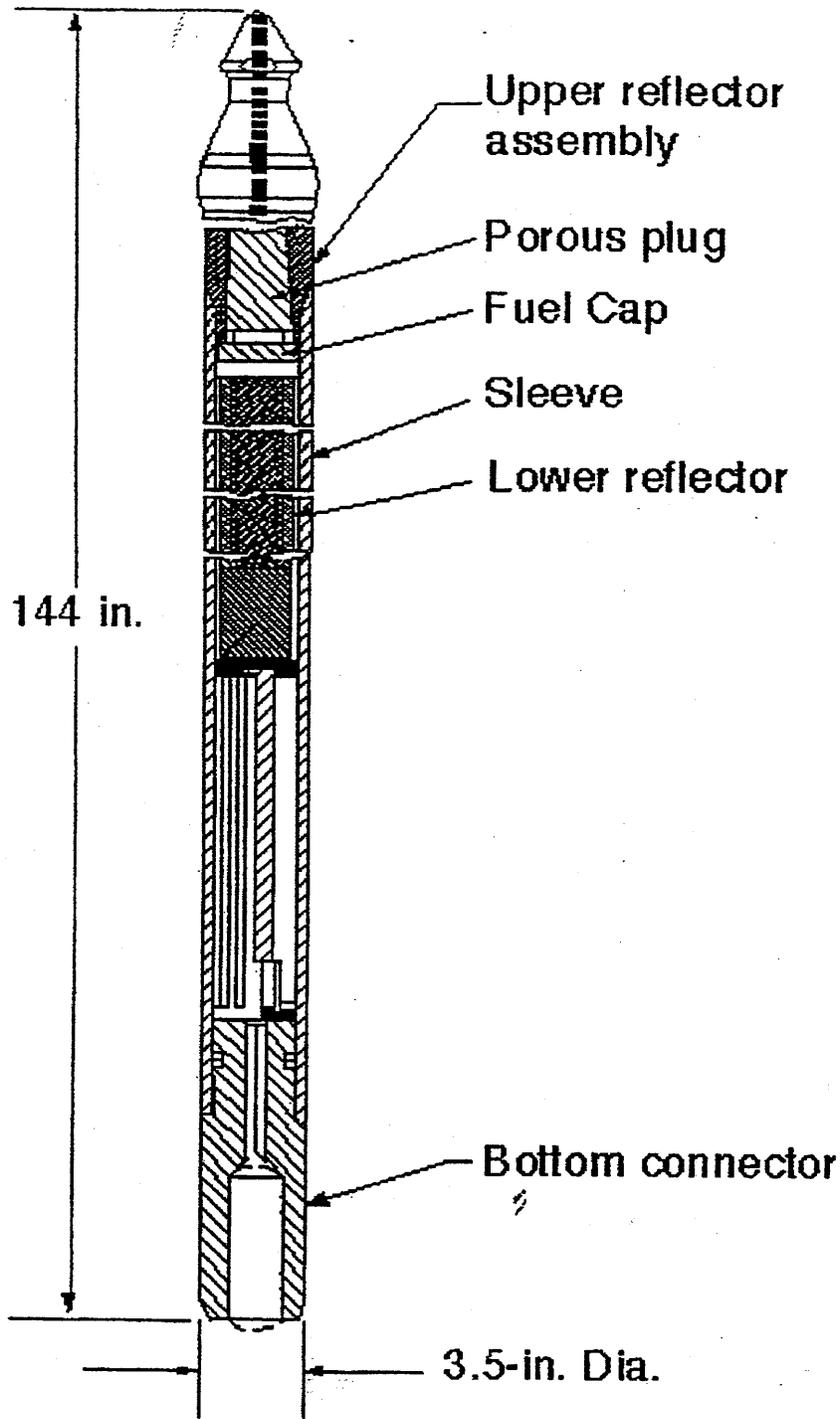
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DRIVER	DATE
1/1/17	2-5-17
TESTER	DATE
1/1/17	2-5-17
INSPECTOR	DATE
1/1/17	2-5-17
APPROVER	DATE
1/1/17	2-5-17

FOR U.S. DEPARTMENT OF ENERGY
 IDAHO CHEMICAL PROCESSING PLANT
 IDAHO FALLS, IDAHO

INSTALLATION
 CPP-003
 MODIFY CASK CRADLE ON TRAILER #71808
 PLAN, ELEVATION & MATL LIST

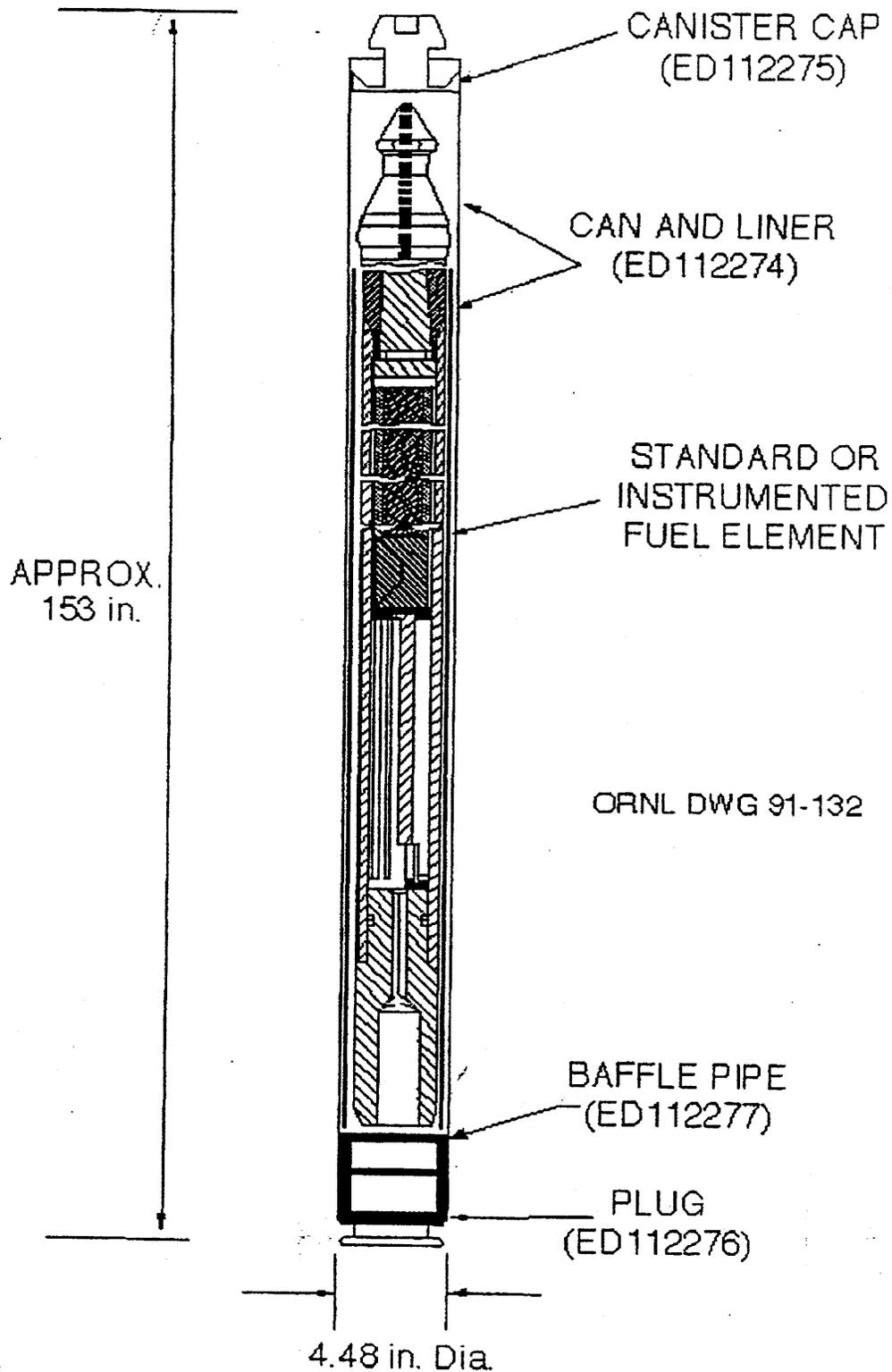
DATE ISSUED: 02-28-17
 SCALE: 1/2" = 1'-0"

JOB NUMBER: 120655
 DRAWING NUMBER: 094563
 REV: 1



Peach bottom assembly

Figure A-14 Standard Peach Bottom Unit 1 Assembly



Peach Bottom Spent Fuel Storage Canister

Figure A-15 Rev. 4

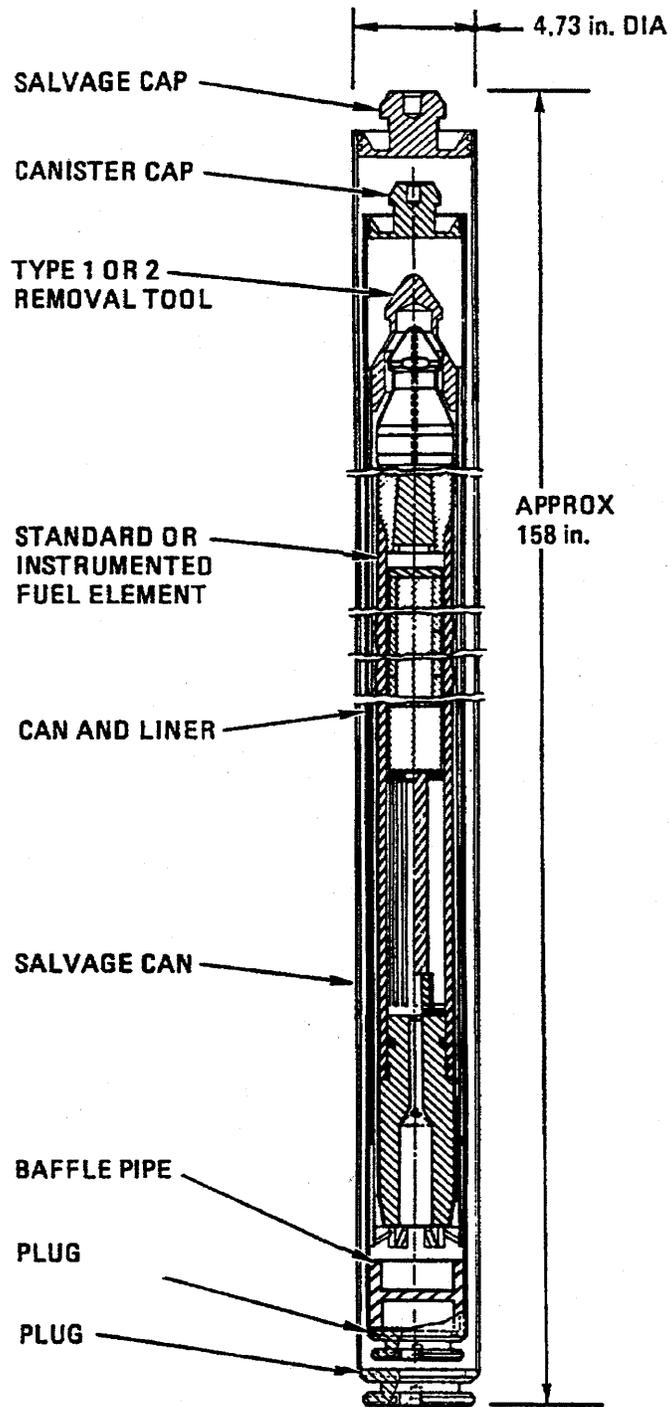


Figure A-16 Aluminum Canister with PB-1 Core 1 Fuel Element with Assembly Removal Tool (with Salvage Can)

Figure A-19

Figure Not Used.

Figure A-20

Figure Not Used.

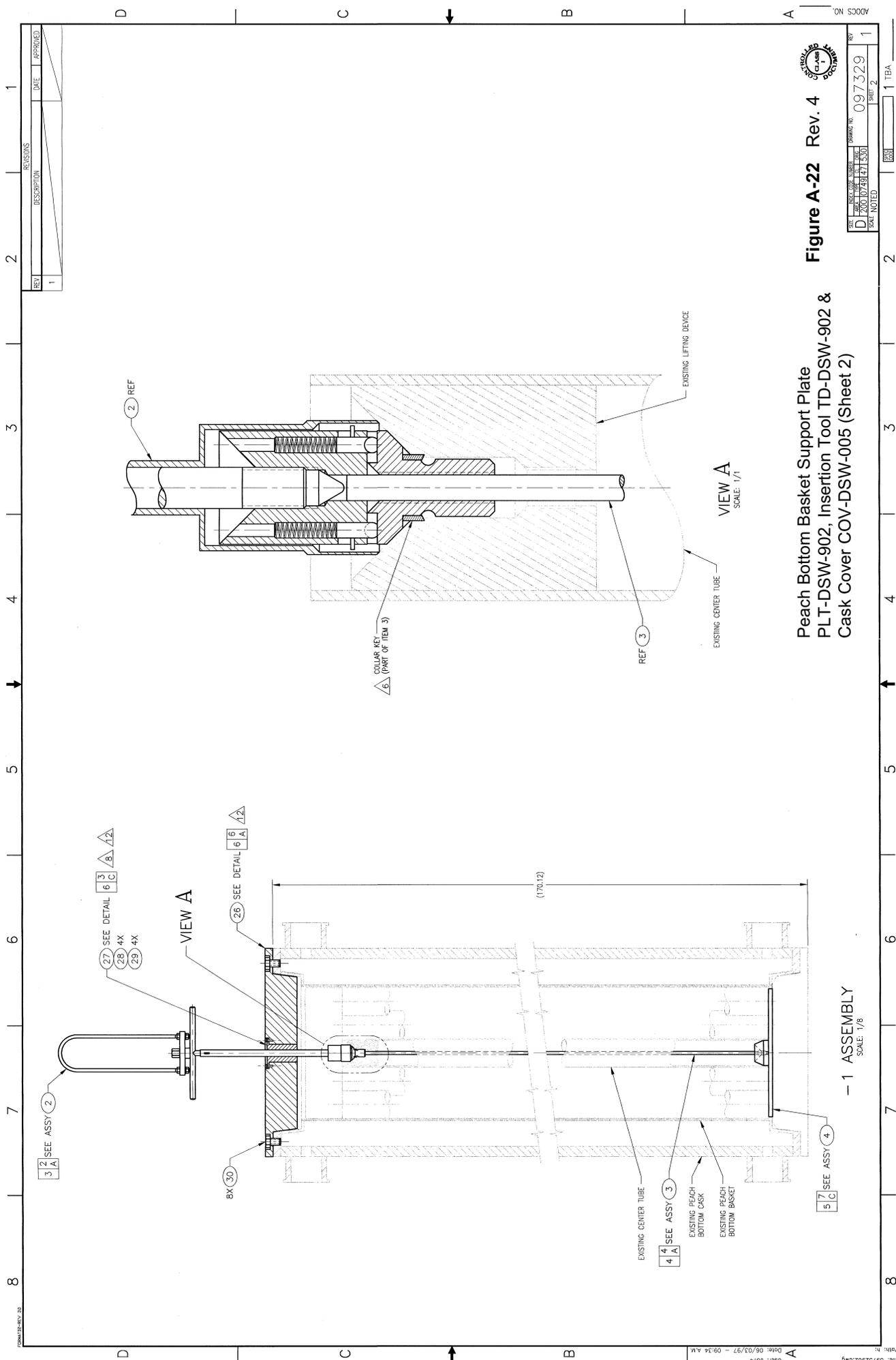


Figure A-22 Rev. 4
Peach Bottom Basket Support Plate
PLT-DSW-902, Insertion Tool TD-DSW-902 &
Cask Cover COV-DSW-005 (Sheet 2)

-1 ASSEMBLY
 SCALE: 1/8

Figure A-22 Rev. 4

REV	DESCRIPTION	DATE	APPROVED
1			

DATE	ISSUE NUMBER	ISSUE NO.	REV
06/03/97	01	097329	1
SCALE	NOTED	SHEET	2

1 2 3 4 5 6 7 8

8 7 6 5 4 3 2 1

REV	DESCRIPTION	DATE	APPROVED
1			

1 2 3 4 5 6 7 8

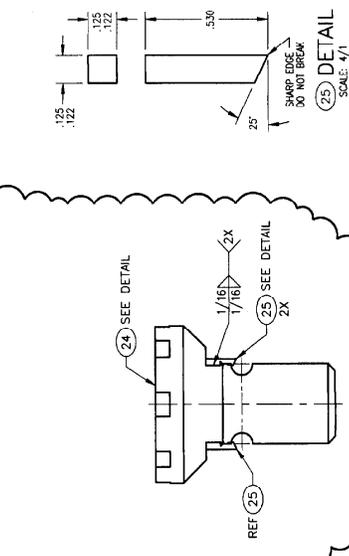
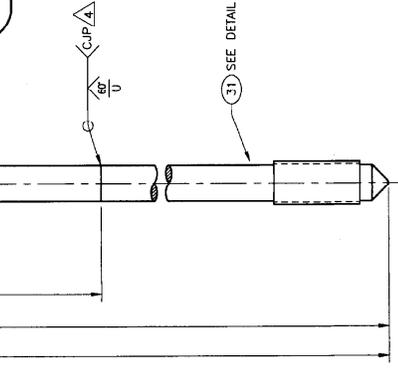
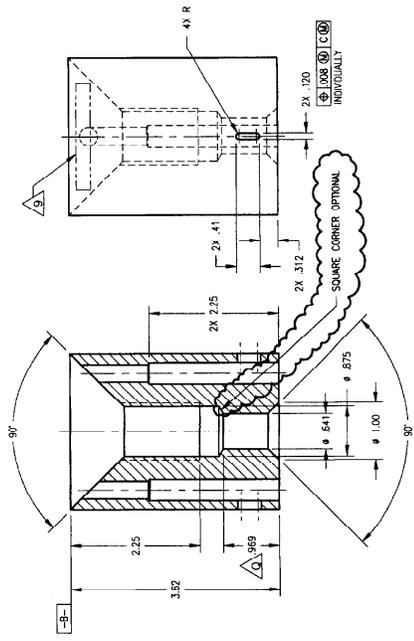
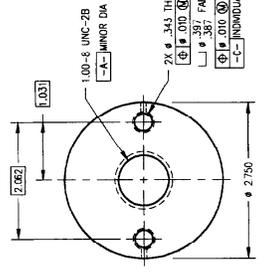
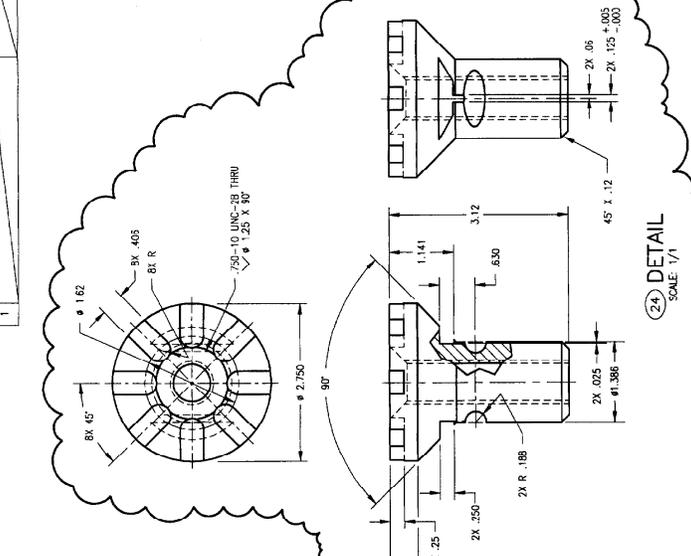
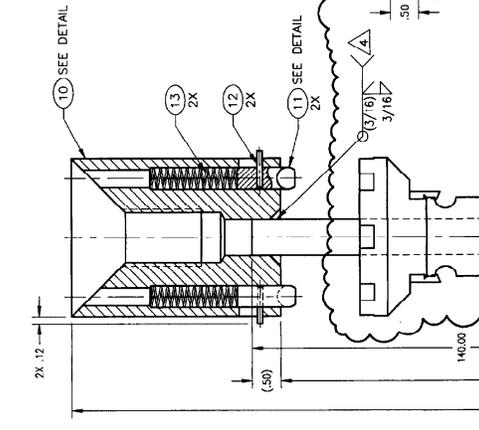
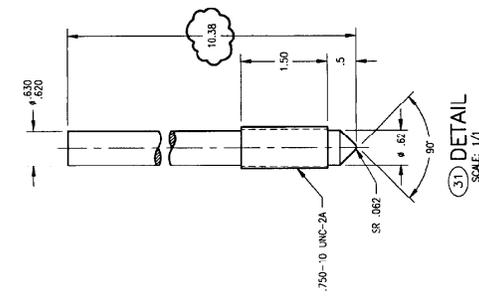
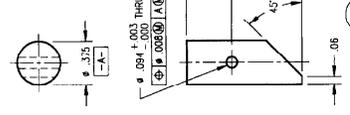


Figure A-24 Rev. 4

REV	DESCRIPTION	DATE	APPROVED
1			

-7 ASSEMBLY
SCALE: 1/1

-3 ASSEMBLY
SCALE: 1/1

Peach Bottom Support Plate PLT-DSW-902, Insertion Tool TD-DSW-902 & Cask Cover COV-DSW-005 (Sheet 4)

ADCS NO. 097329
REV. 4
DATE 07/18/13
BY 1 TBA

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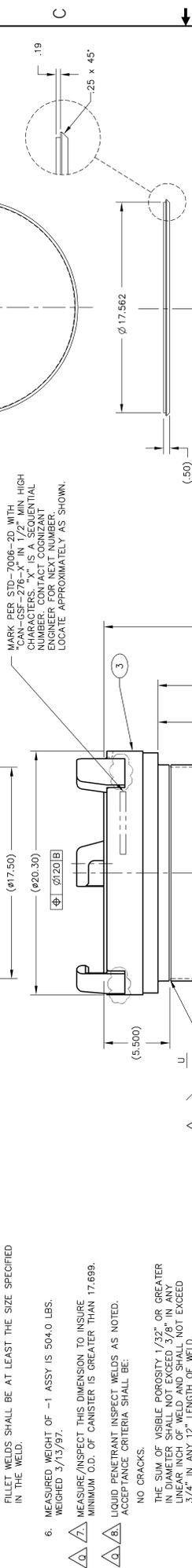
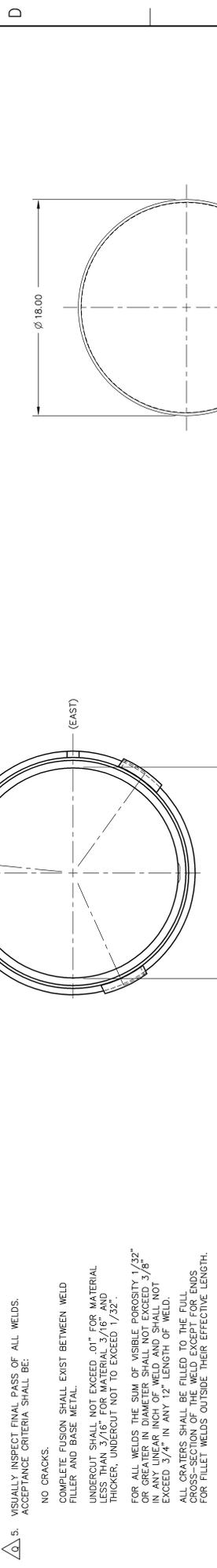
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Figure A-26

Figure Not Used.

REV	DESCRIPTION	DATE	APPROVED
5	SEC DAR B5110	10-3-01	DEJ
6	SEC DAR 96093	10-23-02	DEJ
7	SEC DAR 96936	12/4/02	DEJ



NOTES

- REMOVE ALL BURRS AND SHARP EDGES.
- REMOVED
- PRESERVE AND PROTECT PER STD-7020, CLASS II.
- WELD PER AWS F4.2 OR S6.9 USING ITEM 9, WELD FILLER METAL.
- VISUALLY INSPECT FINAL PASS OF ALL WELDS. ACCEPTANCE CRITERIA SHALL BE:
 - NO CRACKS.
 - COMPLETE FUSION SHALL EXIST BETWEEN WELD FILLER AND BASE METAL.
 - UNDERCUT SHALL NOT EXCEED .01" FOR MATERIAL LESS THAN 3/16" FOR MATERIAL 3/16" AND THICKER; UNDERCUT NOT TO EXCEED 1/32".
 - FOR ALL WELDS THE SUM OF VISIBLE POROSITY 1/32" OR GREATER DIAMETER SHALL NOT EXCEED 3/8" IN ANY LINEAR INCH OF WELD AND SHALL NOT EXCEED 3/4" IN ANY 12" LENGTH OF WELD.
 - ALL CRACKERS SHALL BE FILLED TO THE FULL CROSS-SECTION OF THE WELD EXCEPT FOR ENDS FOR FILLET WELDS OUTSIDE THEIR EFFECTIVE LENGTH.
 - FILLET WELDS SHALL BE AT LEAST THE SIZE SPECIFIED IN THE WELD.
- MEASURED WEIGHT OF -1 ASSY IS 504.0 LBS.
- MEASURE/INSPECT THIS DIMENSION TO INSURE MINIMUM C.D. OF CANISTER IS GREATER THAN 17.699.
- LIQUID PENETRANT INSPECT WELDS AS NOTED. ACCEPTANCE CRITERIA SHALL BE:
 - NO CRACKS.
 - THE SUM OF VISIBLE POROSITY 1/32" OR GREATER IN DIAMETER SHALL NOT EXCEED 3/8" IN ANY LINEAR INCH OF WELD AND SHALL NOT EXCEED 3/4" IN ANY 12" LENGTH OF WELD.
- REMOVED
- MANUFACTURER OR SUPPLIER MUST PROVIDE ACTUAL CERTIFIED MATERIAL TEST REPORTS (CMTR'S) WHICH CERTIFY THAT THE MATERIAL HAS BEEN INSPECTED AND TESTED IN ACCORDANCE WITH THE REQUIREMENTS OF THE SPECIFICATION AND THAT THE RESULTS OF THE INSPECTION AND TESTING MEET THE REQUIREMENTS OF THE SPECIFICATION FOR THE SPECIFIC GRADE. THE MANUFACTURER OR SUPPLIER MUST MARK THE HEAT NUMBER ON THE CMTR'S AND ON THE MATERIAL.
- MANUFACTURER OR INDEPENDENT TESTING LABORATORY MUST PROVIDE ACTUAL CERTIFIED MATERIAL TEST REPORTS (CMTR'S) WHICH CERTIFY THAT THE MATERIAL HAS BEEN INSPECTED AND TESTED IN ACCORDANCE WITH THE REQUIREMENTS OF THE SPECIFICATION AND THAT THE RESULTS OF THE CHEMICAL ANALYSIS MEET THE REQUIREMENTS OF THE SPECIFICATION FOR THE AWS MATERIAL CLASSIFICATION. THE MANUFACTURER OR SUPPLIER MUST FLAG TAG EACH END OF WELD FILLER METAL SHOWING THE AWS MATERIAL CLASSIFICATION AND THE HEAT NUMBER. THE HEAT NUMBER MUST APPEAR ON THE CMTR'S.
- FINAL ASSEMBLY SHALL MEET THE CLEANLINESS REQUIREMENTS OF STD-7022 ACCEPTANCE LEVEL D.

NO CRACKS.

COMPLETE FUSION SHALL EXIST BETWEEN WELD FILLER AND BASE METAL.

UNDERCUT SHALL NOT EXCEED .01" FOR MATERIAL LESS THAN 3/16" FOR MATERIAL 3/16" AND THICKER; UNDERCUT NOT TO EXCEED 1/32".

FOR ALL WELDS THE SUM OF VISIBLE POROSITY 1/32" OR GREATER DIAMETER SHALL NOT EXCEED 3/8" IN ANY LINEAR INCH OF WELD AND SHALL NOT EXCEED 3/4" IN ANY 12" LENGTH OF WELD.

ALL CRACKERS SHALL BE FILLED TO THE FULL CROSS-SECTION OF THE WELD EXCEPT FOR ENDS FOR FILLET WELDS OUTSIDE THEIR EFFECTIVE LENGTH.

FILLET WELDS SHALL BE AT LEAST THE SIZE SPECIFIED IN THE WELD.

MEASURED WEIGHT OF -1 ASSY IS 504.0 LBS.

MEASURE/INSPECT THIS DIMENSION TO INSURE MINIMUM C.D. OF CANISTER IS GREATER THAN 17.699.

LIQUID PENETRANT INSPECT WELDS AS NOTED. ACCEPTANCE CRITERIA SHALL BE:

NO CRACKS.

THE SUM OF VISIBLE POROSITY 1/32" OR GREATER IN DIAMETER SHALL NOT EXCEED 3/8" IN ANY LINEAR INCH OF WELD AND SHALL NOT EXCEED 3/4" IN ANY 12" LENGTH OF WELD.

REMOVED

MANUFACTURER OR SUPPLIER MUST PROVIDE ACTUAL CERTIFIED MATERIAL TEST REPORTS (CMTR'S) WHICH CERTIFY THAT THE MATERIAL HAS BEEN INSPECTED AND TESTED IN ACCORDANCE WITH THE REQUIREMENTS OF THE SPECIFICATION AND THAT THE RESULTS OF THE CHEMICAL ANALYSIS MEET THE REQUIREMENTS OF THE SPECIFICATION FOR THE AWS MATERIAL CLASSIFICATION. THE MANUFACTURER OR SUPPLIER MUST FLAG TAG EACH END OF WELD FILLER METAL SHOWING THE AWS MATERIAL CLASSIFICATION AND THE HEAT NUMBER. THE HEAT NUMBER MUST APPEAR ON THE CMTR'S.

FINAL ASSEMBLY SHALL MEET THE CLEANLINESS REQUIREMENTS OF STD-7022 ACCEPTANCE LEVEL D.

Figure A-27 Rev. 4

ITEM	QTY	RECD	CAL	SAFETY	PART OR IDENTIFYING NO.	DESCRIPTION/MATERIAL/SPECIFICATION	UNIT	FINO
AR	SS					WELD FILLER METAL	ER 308L, AMS A5.9	9
								8
								7
AR	SS					CANISTER BODY	PIPE, 18 SCHED 10S (18B WALL) 304L SST, ASTM A312	6
1	SS			-5		CANISTER BOTTOM	PLATE, 1/2 THK 304L SST, ASTM A240	5
								4
1	SS			453319-1		CANISTER TOP ASSEMBLY		3
								2
								1
								1

Locitec
 Aero Technologies Company
 CPP-603 IFSF
 LIGHTER WEIGHT
 STORAGE CANISTER ASSEMBLY
 CAN-GSF-276-X
 DRAWING NO. 453318
 DATE 8/19/96
 SCALE 1/A & NOTED
 SHEET 1 OF 1

REV	STATUS	DESCRIPTION	REVISIONS	DATE	APPROVED
3	SEE DWR 67632			8-23-00	DEJ
4	ADDED INTEC AREA PLANT CODE AND HS			10-23-02	DEJ
5	SEE DWR 100952			WH, 6-12-03	DEJ

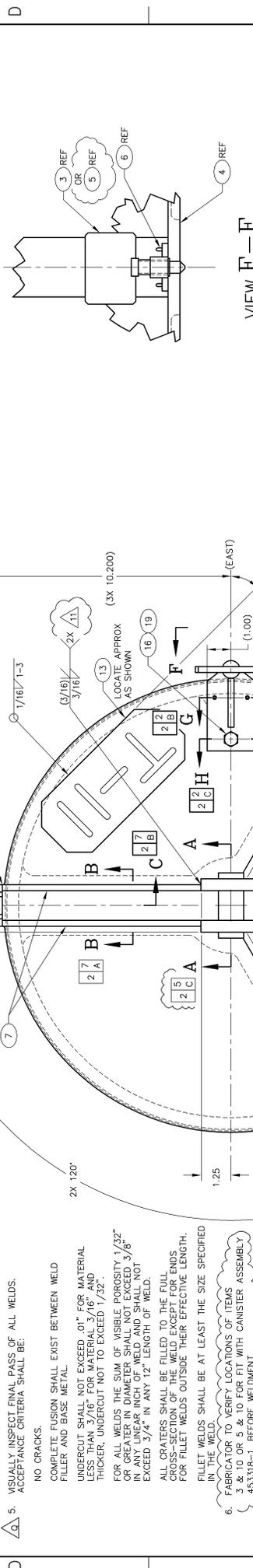
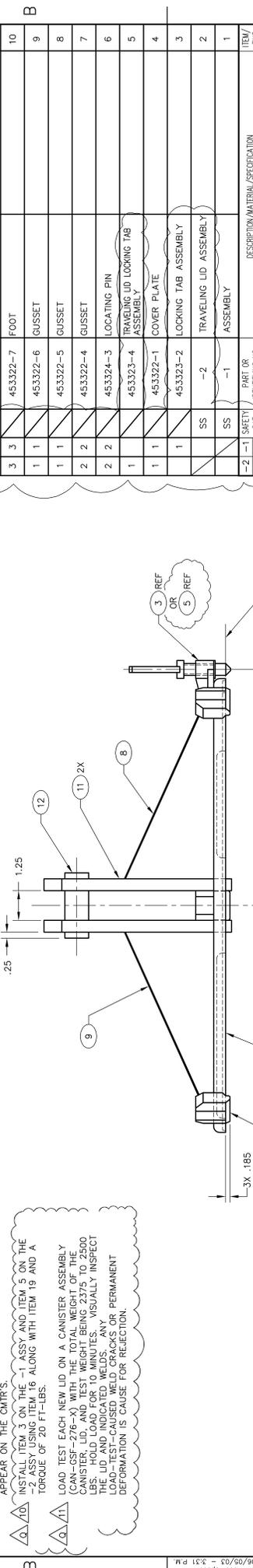


Figure A-28 Rev. 4
 -1 ASSEMBLY (SHOWN)
 -2 ASSEMBLY

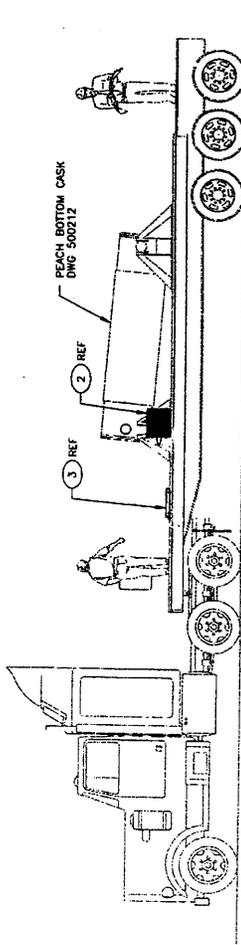
ITEM NO.	QTY	SAFETY	DESCRIPTION, MATERIAL, SPECIFICATION	PART OR IDENTIFYING NO.	DESCRIPTION, MATERIAL, SPECIFICATION	ITEM FINO
19			LOCITITE CORP.			
18			PRO-LOCK	81741		
17			WELD FILLER METAL			
16	1		SCREW	453324-4		
15						
14						
13	1		NAME PLATE	453322-10		
12	1		LIFTING BAR	453322-9		
11	2		LIFTING BAIL	453322-8		
10	3		FOOT	453322-7		
9	1		GUSSET	453322-6		
8	1		GUSSET	453322-5		
7	2		GUSSET	453322-4		
6	2		LOCATING PIN	453324-3		
5	1		TRAVELING LID LOCKING TAB ASSEMBLY	453323-4		
4	1		COVER PLATE	453322-1		
3	1		LOCKING TAB ASSEMBLY	453323-2		
2			TRAVELING LID ASSEMBLY	-2		
1			ASSEMBLY	-1		



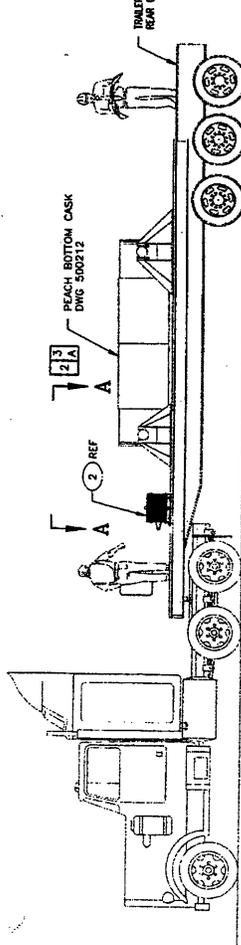
REV	STATUS	DESCRIPTION	REVISIONS	DATE	APPROVED
3	SEE DWR 67632			8-23-00	DEJ
4	ADDED INTEC AREA PLANT CODE AND HS			10-23-02	DEJ
5	SEE DWR 100952			WH, 6-12-03	DEJ

NOTES:

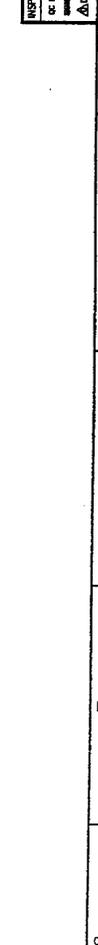
- △ -2 ASSEMBLY (SADDLE ASSEMBLY) SHALL BE LAYED IN THE FOLLOWING MANNER:
 - 1. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 2. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 3. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 4. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 5. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 6. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 7. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 8. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 9. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 10. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 11. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 12. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 13. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 14. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 15. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 16. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 17. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 18. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 19. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 20. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 21. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 22. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 23. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 24. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 25. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 26. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 27. 1/4" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
 - 28. 1/2" THICK PLYWOOD SHEETED USING ITEM 5 (SCREWS).
- △ SEAL AROUND ALL EDGES BETWEEN ITEM 3 AND TRAILER BED AND AROUND ALL ITEMS 12 & 13 (LOCK WASHER AND NUT).
- △ INSTALL ITEM 11 (NUT) WITH HEAD UNDER TRAILER. ITEM 15 (LOCK WASHER) SHALL BE ON NUT SIDE. TORQUE ITEM 12 (NUT) TO 4822 FT/LBS.
- △ INSTALL ITEM 13 (NUT) WITH HEAD TOWARD INSIDE OF ITEM 3. ITEM 15 (LOCK WASHER) AND ITEM 15 (LOCK WASHER) SHALL BE ON NUT SIDE. TORQUE ITEM 8 (NUT) TO 17084 FT/LBS.
- △ FASTEN -2 ASX (SADDLE ASSEMBLY) TO TRAILER BED WITH ITEM 18 (TIE DOWN) WHILE NOT IN USE.
- △ CUT ITEM 7 (ALL THREAD) FLUSH WITH ITEM 12 (NUTS).
- △ ITEM 16 IS SOLD IN PAKS. QTY REQUIRED FOR -2 ASSEMBLY IS 4.
- △ QUANTITY ORDERED WILL BE 2 PAIR.
- △ VISUALLY INSPECT ALL WELDS PER AWS D1.1. ACCEPTANCE CRITERIA SHALL BE PER AWS D1.1 FOR SIMILARLY LOADED STRUCTURES.
- △ STENCIL "500-05M-900" AND "WEIGHT 250 LBS" IN 1/2" HIGH CHARACTERS APPROXIMATELY WHERE SHOWN USING ITEM 23 (STENCIL INK).
- △ WATCH DRILL #156 THRU TRAILER DECK PAINT USING ITEM 3 AS A TEMPLATE.



-1 INSTALLATION
SADDLE USED TO SUPPORT CASK



-1 INSTALLATION
SADDLE IN STOWED POSITION



REV	REV STATUS OF SHEET	DESCRIPTION	REVISIONS	DATE
1	REV	SEE DWG 8182 NO. 1746	KA	4-18-77
2	1	WASHER, FLAT, 3/4		
2	4	814784636		
AR		814784636		
2		-25		
AR		RY108		
AR		20859		
AR		P-1-0445		
AR		6040		
AR				
1		9140755-030		
AR		PL100		
AR		1984		
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AR		90095A132		
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REV	REV STATUS OF SHEET	DESCRIPTION	REVISIONS	DATE
1	REV	SEE DWG 8182 NO. 1746	KA	4-18-77
2	1	WASHER, FLAT, 3/4		
2	4	814784636		
AR		814784636		
2		-25		
AR		RY108		
AR		20859		
AR		P-1-0445		
AR		6040		
AR				
1		9140755-030		
AR		PL100		
AR		1984		
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AR				
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INSPECTION REQUIREMENTS

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QC INSPECTED: _____

QC APPROVED: _____

DATE: _____

BY: _____

REVISIONS:

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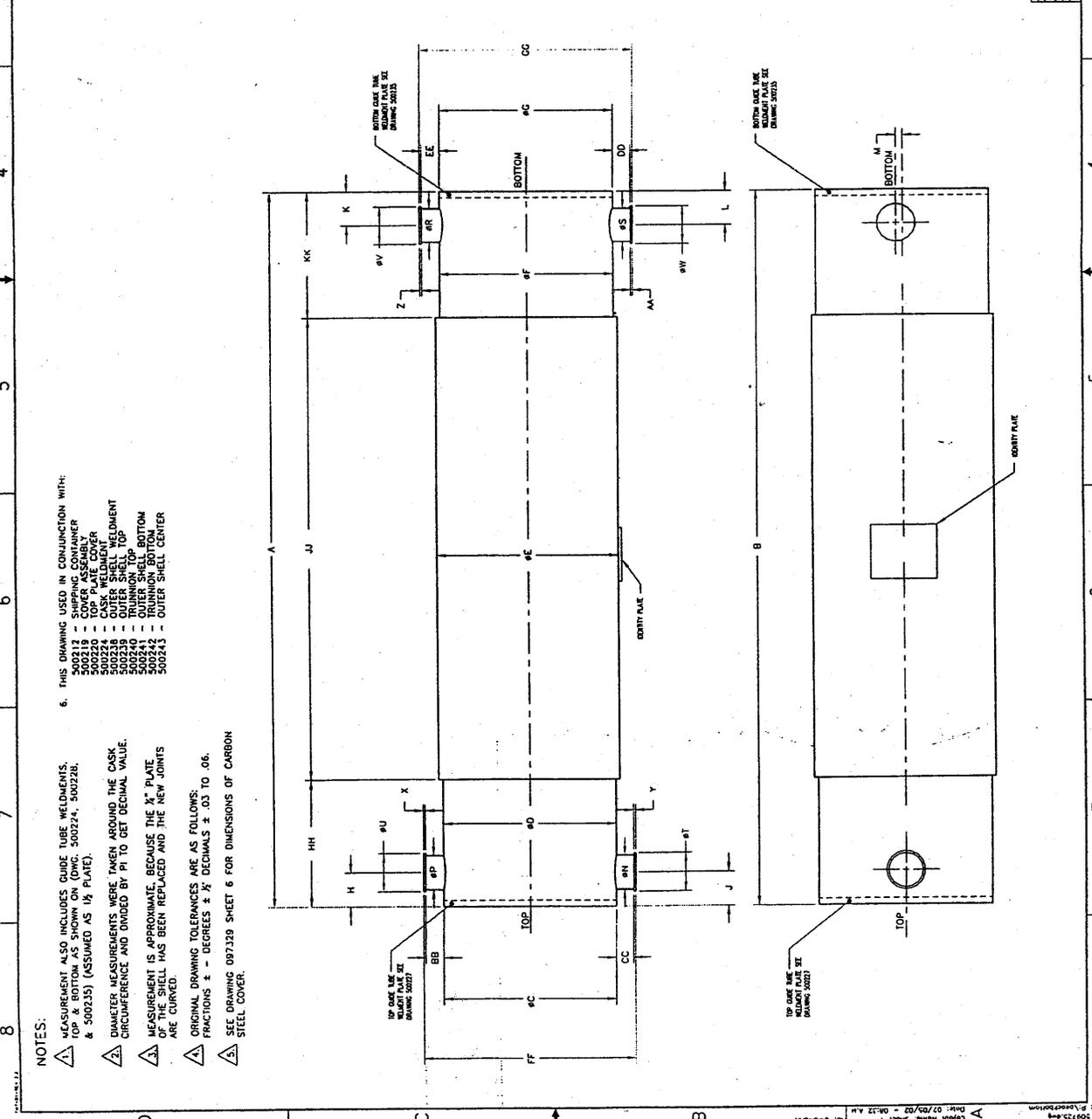
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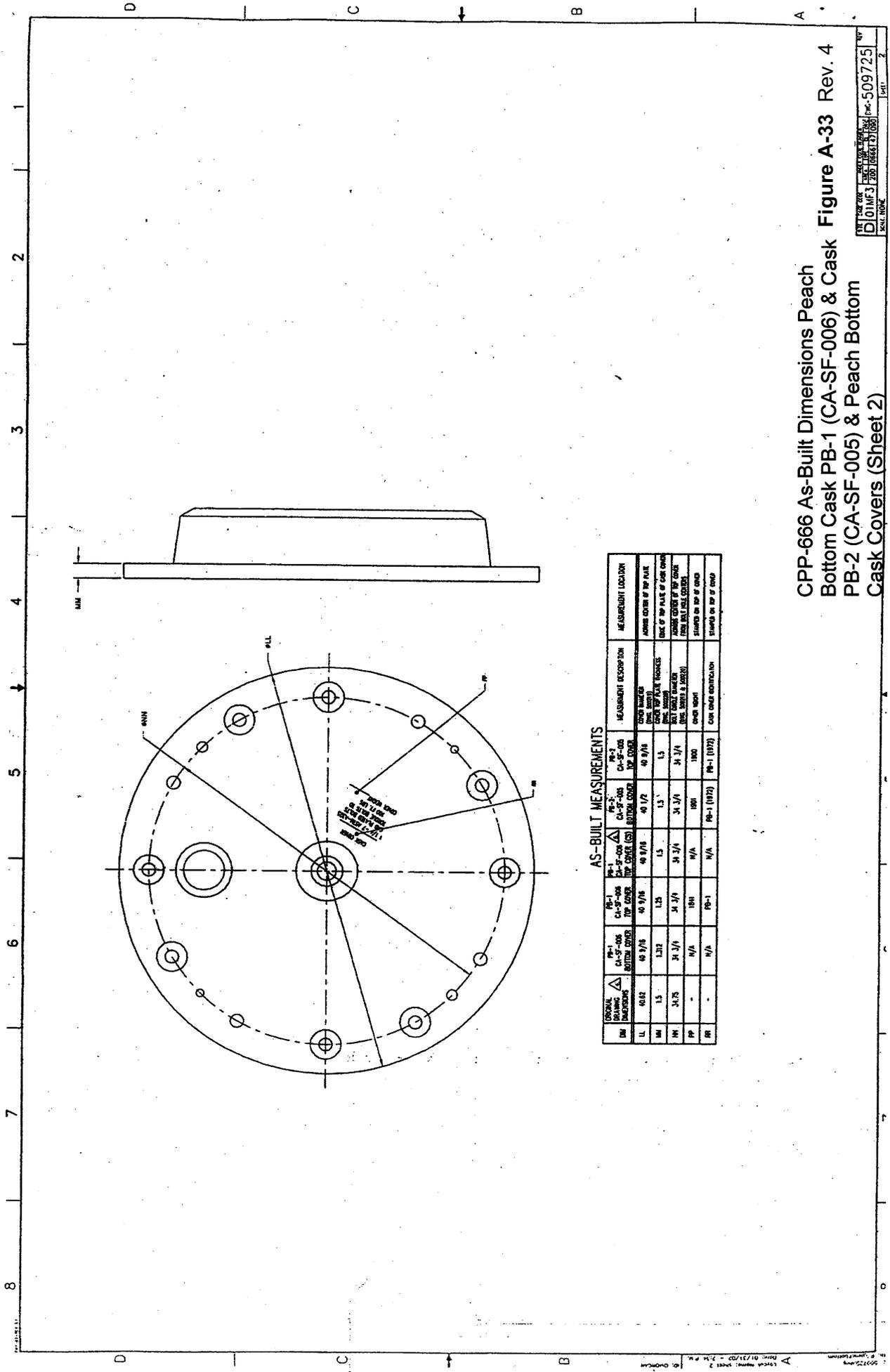
Figure A-30 Rev. 4

ORIGINAL DIMENSIONS	CASK PR-1 CA-SF-006	CASK PR-2 CA-SF-008	MEASUREMENT DESCRIPTION	MEASUREMENT LOCATION
A	170.12	169 3/4	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
B	170.12	169 3/4	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
C	40.00	40.64	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
D	40.82	40.58	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
E	42.82	42.75	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
F	40.82	40.58	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
G	40.82	40.58	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
H	8.5	8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
I	8.5	8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
J	8.5	8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
K	8.5	8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
L	8.5	8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
M	1.5	1	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
N	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
O	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
P	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
Q	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
R	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
S	8	7 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
T	9	8 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
U	9	8 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
V	9	8 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
W	9	8 15/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
X	5	5	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
Y	5	5	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
Z	5	5	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
AA	5	5	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
BB	4	4 1/8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
CC	4	4 1/8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
DD	4	4 1/8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
EE	5	4 7/8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
FF	50	50	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
GG	50	50	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
HH	25.56	30 3/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
II	110.00	109 1/8	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
JJ	25.56	30 3/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER
KK	25.56	30 3/16	TOP FLANGE FROM CASK CENTER	TOP FLANGE FROM CASK CENTER

AS-BUILT MEASUREMENTS



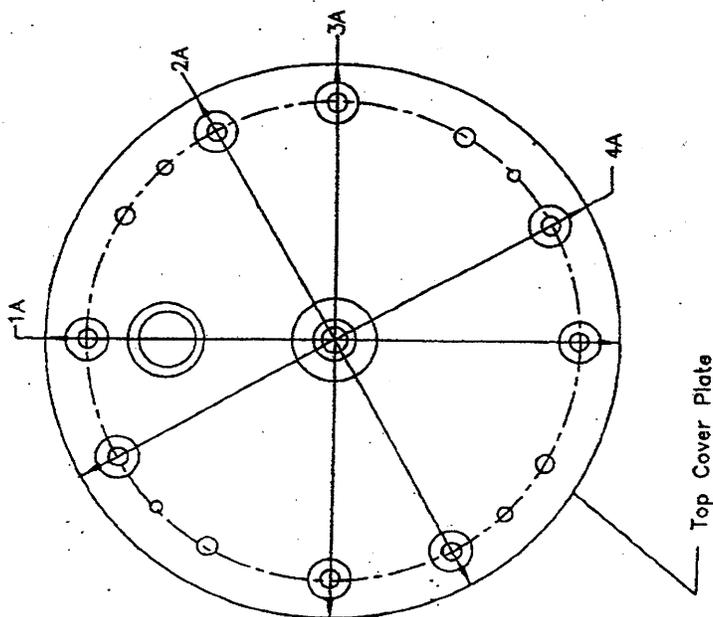
NOTES:
 △ MEASUREMENT ALSO INCLUDES GUIDE TUBE WELDMENTS, TOP & BOTTOM AS SHOWN ON DWG. 500224, 500228, & 500235 (ASSUMED AS 1/2 PLATE).
 △ DIAMETER MEASUREMENTS WERE TAKEN AROUND THE CASK CIRCUMFERENCE AND DIVIDED BY PI TO GET DECIMAL VALUE.
 △ MEASUREMENT IS APPROXIMATE, BECAUSE THE 1/2 PLATE ARE CURVED.
 △ ORIGINAL DRAWING TOLERANCES ARE AS FOLLOWS:
 △ FRACTIONS ± - DECIMALS ± 1/32 TO .06.
 △ SEE DRAWING 087329 SHEET 6 FOR DIMENSIONS OF CARBON STEEL COVER.



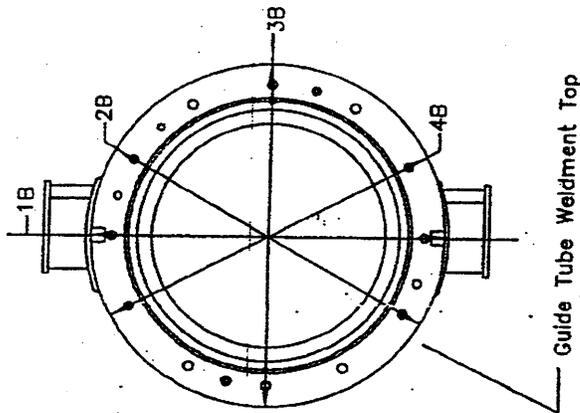
AS-BUILT MEASUREMENTS

ORIGINAL DIMENSIONS	PP-1 CA-SF-005 TOP COVER	PP-2 CA-SF-006 BOTTOM COVER	PP-3 CA-SF-008 TOP COVER	MEASUREMENT DESCRIPTION	MEASUREMENT LOCATION
LL	40 9/16	40 9/16	40 1/2	OUTER DIAMETER	INNER CIRCLE OF TOP PLATE
MM	1.312	1.5	1.5	INNER DIAMETER	OUTER CIRCLE OF TOP PLATE
NN	3/4	3/4	3/4	DIAMETER OF HOLE	FACE OF THE HOLE OF TOP COVER
PP	N/A	N/A	1.875	OFFSET FROM CENTER	FACE OF THE HOLE OF TOP COVER
PP	N/A	N/A	1.875	OFFSET FROM CENTER	FACE OF THE HOLE OF TOP COVER

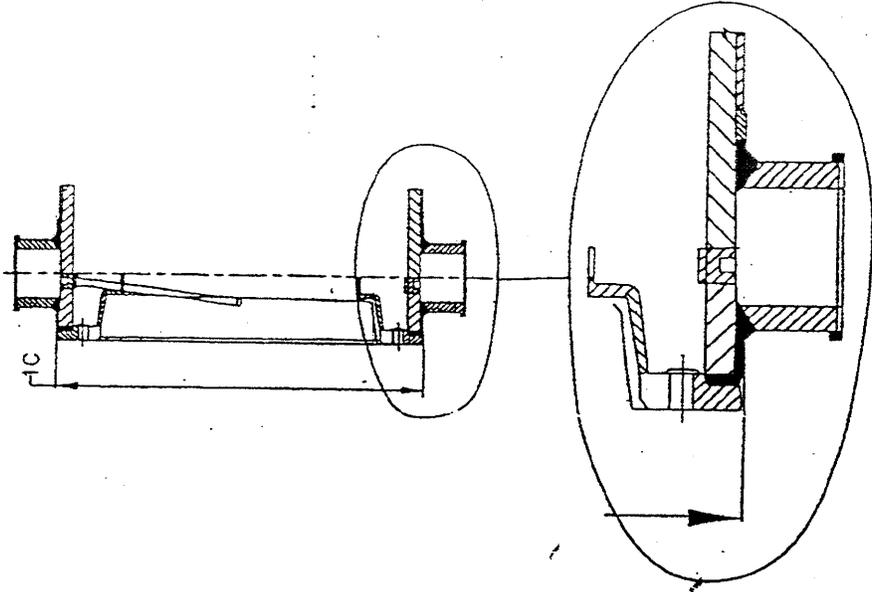
CPP-666 As-Built Dimensions Peach
 Bottom Cask PB-1 (CA-SF-006) & Cask
 PB-2 (CA-SF-005) & Peach Bottom
 Cask Covers (Sheet 2)



Top Cover Plate



Guide Tube Weldment Top



Note: All As-Built Dimensions Shall Be Within $\pm 1/32$ Inches

DIM	ORIGINAL DRAWING DIMENSIONS	PB-1 TOP COVER CA-SF-006	PB-1 TOP COVER CA-SF-008(CS)	PB-2 TOP COVER CA-SF-005
1A	40.62	40.617	40.625	40.619
2A	40.62	40.617	40.625	40.619
3A	40.62	40.616	40.616	40.619
4A	40.62	40.615	40.615	40.615

DIM	ORIGINAL DRAWING DIMENSIONS	PB-1 CASK	PB-2 CASK
1B	40.67	40.67	40.670
2B	40.62	40.625	40.610
3B	40.62	40.62	40.610
4B	40.62	40.619	40.590

DIM	ORIGINAL DRAWING DIMENSIONS	PB-1 Cask	PB-2 Cask
1C	41	40.751	40.520

Figure A-34 Rev. 4

As-Built Dimensions Top
Cover Plate Peach Bottom

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ITEM NO	DESCRIPTION	QTY	UNIT	REV	DATE	BY	CHKD	APP'D
1	LINER BODY ASSY GR 1 DWS 1511E 12	1	EA	1				
2	HEAD ASSY GR 1 DWS 1511E 12	1	EA	1				
3	SPACER GR 1 DWS 1511E 12	1	EA	1				
4	CRUSH PLATE ASSY GR 1 DWS 2155F 12	1	EA	1				
5	SPACER GR 1 DWS 1511E 12	1	EA	1				
6	CRUSH PLATE ASSY GR 1 DWS 2155F 12	1	EA	1				
7	SPACER GR 1 DWS 1511E 12	1	EA	1				
8	CRUSH PLATE ASSY GR 1 DWS 2155F 12	1	EA	1				
9	GUANO RING ASSY GR 1 DWS 2155F 12	1	EA	1				
10	GUANO RING ASSY GR 1 DWS 2155F 12	1	EA	1				
11	GUANO RING ASSY GR 1 DWS 2155F 12	1	EA	1				
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49	GUANO RING ASSY GR 1 DWS 2155F 12	1	EA	1				
50	GUANO RING ASSY GR 1 DWS 2155F 12	1	EA	1				

LIST OF MATERIAL

ORDERING INFORMATION

1. LINER BODY ASSY GR 1 DWS 1511E 12

2. HEAD ASSY GR 1 DWS 1511E 12

3. SPACER GR 1 DWS 1511E 12

4. CRUSH PLATE ASSY GR 1 DWS 2155F 12

5. SPACER GR 1 DWS 1511E 12

6. CRUSH PLATE ASSY GR 1 DWS 2155F 12

7. SPACER GR 1 DWS 1511E 12

8. CRUSH PLATE ASSY GR 1 DWS 2155F 12

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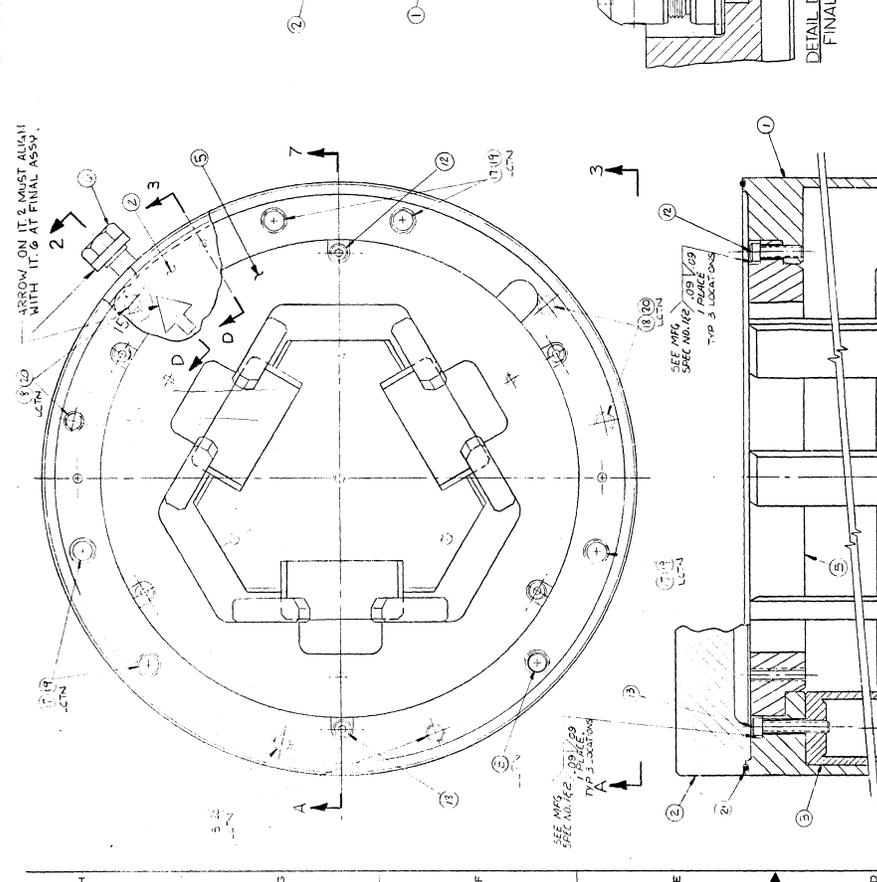
46. GUANO RING ASSY GR 1 DWS 2155F 12

47. GUANO RING ASSY GR 1 DWS 2155F 12

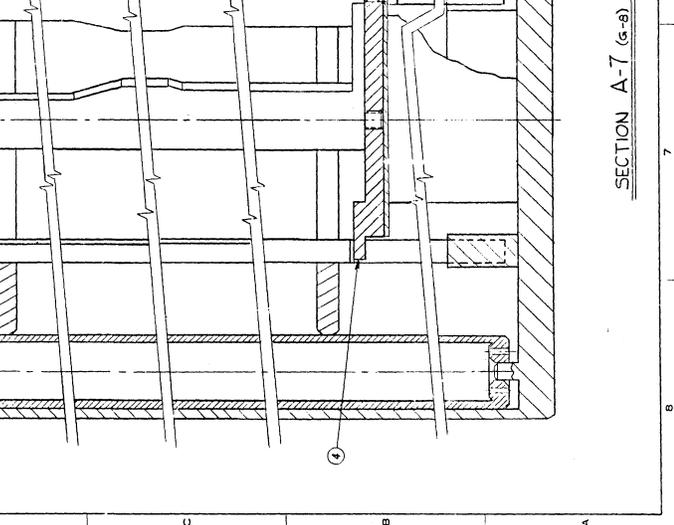
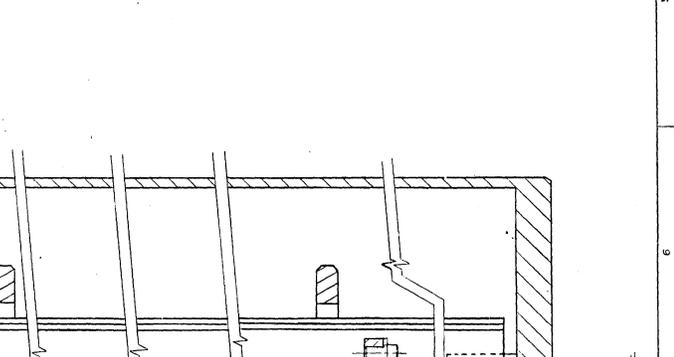
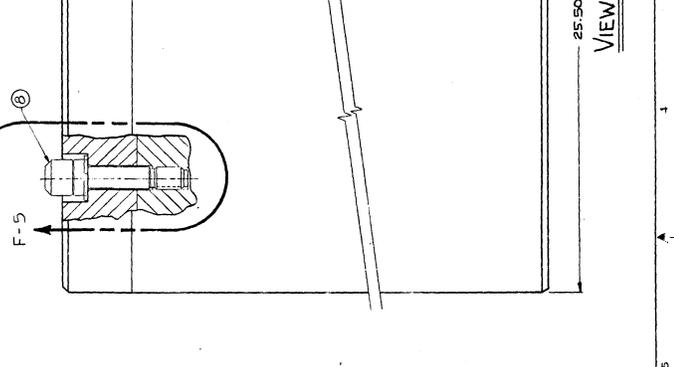
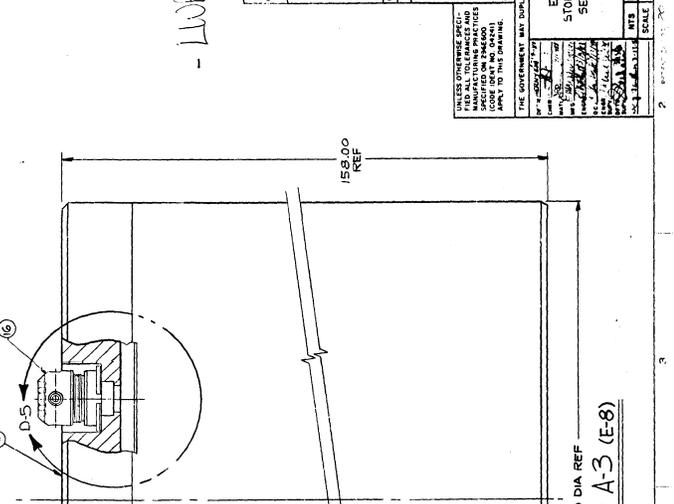
48. GUANO RING ASSY GR 1 DWS 2155F 12

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THE GOVERNMENT MAY DUPLICATE, USE, AND DISCLOSE THIS DRAWING FOR ANY PURPOSE.

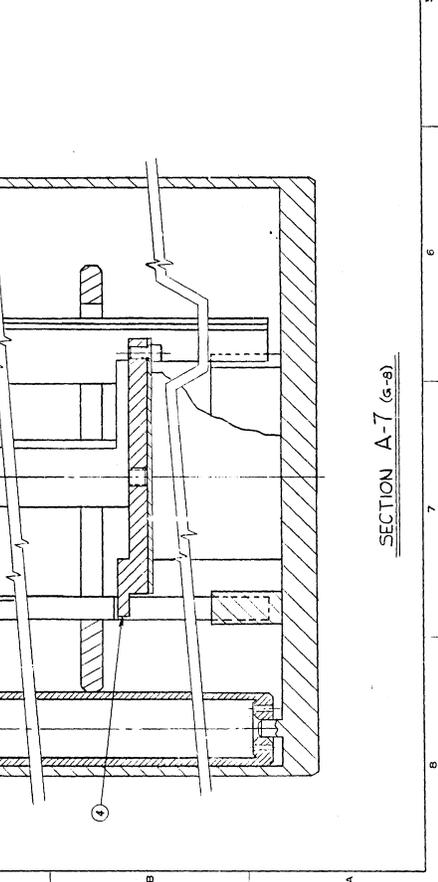
DATE: 12/12/11

CLASSIFICATION: UNCLASSIFIED

WESTINGHOUSE ELECTRIC CORPORATION
 400 WESTINGHOUSE AVENUE
 PITTSBURGH, PA 15222-4100
 PHONE: 412.765.1000
 FAX: 412.765.1001
 WWW: WWW.WESTINGHOUSE.COM

SCALE: AS SHOWN

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SECTION A-7 (G-8)

